

EXHIBIT B

License Amendment Request Dated April 9, 1987

Docket No. 50-263

License No. DPR-22

Exhibit B consists of marked up pages of the Monticello Technical Specifications showing the proposed changes as listed below:

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3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

4.0 SURVEILLANCE REQUIREMENTS

- A. The surveillance requirements of this section shall be met. Each surveillance requirements shall be performed at the specified times except as allowed in B and C below
- B. Specific time intervals between tests may be extended up to 25% of the surveillance interval to accomodate normal test schedules with the exception that, the intervals between tests scheduled for refueling shutdowns shall not exceed two years.
- C. Whenever the plant condition is such that a system or component is not required to be operable the surveillance testing associated with that system or component may be discontinued. Discontinued surveillance tests shall be resumed less than one test interval before establishing plant conditions requiring operability of the associated system or component.

once each operating cycle or tests scheduled

### 3.0 LIMITING CONDITIONS FOR OPERATION

#### II. Snubbers

1. Except as permitted below, all safety related snubbers shall be operable whenever the supported system is required to be Operable.
2. With one or more snubbers made or found to be inoperable for any reason when Operability is required, within 72 hours:
  - a. Replace or restore the inoperable snubbers to Operable status and perform an engineering evaluation or inspection of the supported components, or
  - b. Determine through engineering evaluation that the as-found condition of the snubber had no adverse effect on the supported components and that they would retain their structural integrity in the event of design basis seismic event, or
  - c. Declare the supported system inoperable and take the action required by the Technical Specifications for inoperability of that system.

### 4.0 SURVEILLANCE REQUIREMENTS

#### II. Snubbers

The following surveillance requirements apply to all safety related snubbers.

1. Visual inspection of snubbers shall be conducted in accordance with the following schedule:

No. of Snubbers Found Inoperable per Inspection Period	Next Required Inspection Period
0	<del>18 months ± 25%</del>
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
5,6,7	62 days ± 25%
8 or more	31 days ± 25%

ONCE Each Operating Cycle

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized in two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

### 3.0 LIMITING CONDITIONS FOR OPERATION

### 4.0 SURVEILLANCE REQUIREMENTS

3. Functional testing of snubbers shall be conducted at least once ~~per 18 months~~ <sup>each operating cycle</sup> ~~± 25%~~ during cold shutdown. Ten percent of the total number of each brand of snubber shall be functionally tested either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria in Specification 4.6.H.4 below, an additional ten percent of that brand shall be functionally tested until no more failures are found or all snubbers of that brand have been tested.

The representative sample selected for functional testing shall include the various configurations, operating environments, and the range of size and capacity of the snubbers.

In addition to the regular sample and specified re-samples, snubbers which failed the previous functional test shall be retested during the next test period if they were reinstalled as a safety-related snubber. If a spare snubber has been installed in place of a failed safety related snubber, it shall be tested during the next period.

If any snubber selected for functional testing either fails to lockup or fails to move (i.e. frozen in place) the cause shall be evaluated and if caused by manufacturer or design deficiency, all snubbers of the same design subject to the same defect shall be functionally tested.

### 3.0 LIMITING CONDITIONS FOR OPERATION

- d. During reactor isolation conditions the reactor pressure vessel shall be depressurized to  $< 200$  psig at normal cooldown rates if the suppression pool temperature exceeds  $120^{\circ}\text{F}$ .
- e. The suppression chamber water volume shall be  $\geq 68,000$  and  $\leq 72,910$  cubic feet.
- f. Two channels of torus water level instrumentation shall be operable. From and after the date that one channel is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding 30 days unless such channel is sooner made operable. If both channels are made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding six hours unless at least one channel is sooner made operable.

#### 2. Primary Containment Integrity

Primary containment integrity, as defined in Section 1, shall be maintained at all times when the reactor is critical or when the reactor water temperature is above  $212^{\circ}\text{F}$  and fuel is in the reactor vessel except while performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed  $5 \text{ Mw(t)}$ .

\*The second test of the second 10-year service period may be conducted during the 1989 refueling outage.

### 4.0 SURVEILLANCE REQUIREMENTS

- d. Whenever there is indication of relief valve operation with a suppression pool temperature  $\geq 160^{\circ}\text{F}$  and the primary coolant system pressure  $> 200$  psig, an extended visual examination of the suppression chamber shall be conducted before resuming power operation.
  - e. The suppression chamber water volume shall be checked once per day.
  - f. The suppression chamber water volume indicators shall be calibrated semi-annually.
- #### 2. Primary Containment Integrity
- a. Integrated Primary Containment Leak Test (IPCLT)

The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- 1. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at  $40 \pm 10$  month intervals\* during shutdown at  $P_1$  (41 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant in-service inspection.

~~\*The third test of the first 10 year service period shall be conducted during the 1980 refueling shutdown. The first test of the second 10 year period shall be conducted during the 1984 refueling shutdown.~~

EXHIBIT C

License Amendment Request Dated April 9, 1987

Docket No. 50-263

License No. DPR-22

Exhibit C consists of retyped pages of the Monticello Technical Specifications showing the proposed changes as listed below:

Page

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3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

4.0 SURVEILLANCE REQUIREMENTS

- A. The surveillance requirements of this section shall be met. Each surveillance requirements shall be performed at the specified times except as allowed in B and C below
- B. Specific time intervals between tests may be extended up to 25% of the surveillance interval to accomodate normal test schedules with the exception that, the intervals between tests scheduled once each operating cycle or tests scheduled for refueling shutdowns shall not exceed two years.
- C. Whenever the plant condition is such that a system or component is not required to be operable the surveillance testing associated with that system or component may be discontinued. Discontinued surveillance tests shall be resumed less than one test interval before establishing plant conditions requiring operability of the associated system or component.

### 3.0 LIMITING CONDITIONS FOR OPERATION

#### H. Snubbers

1. Except as permitted below, all safety related snubbers shall be operable whenever the supported system is required to be Operable.
2. With one or more snubbers made or found to be inoperable for any reason when Operability is required, within 72 hours:
  - a. Replace or restore the inoperable snubbers to Operable status and perform an engineering evaluation or inspection of the supported components, or
  - b. Determine through engineering evaluation that the as-found condition of the snubber had no adverse effect on the supported components and that they would retain their structural integrity in the event of design basis seismic event, or
  - c. Declare the supported system inoperable and take the action required by the Technical Specifications for inoperability of that system.

### 4.0 SURVEILLANCE REQUIREMENTS

#### H. Snubbers

The following surveillance requirements apply to all safety related snubbers.

1. Visual inspection of snubbers shall be conducted in accordance with the following schedule:

<u>No. of Snubbers Found Inoperable per Inspection Period</u>	<u>Next Required Inspection Period</u>
0	Once each operating cycle
1	12 months $\pm$ 25%
2	6 months $\pm$ 25%
3,4	124 days $\pm$ 25%
5,6,7	62 days $\pm$ 25%
8 or more	31 days $\pm$ 25%

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized in two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

### 3.0 LIMITING CONDITIONS FOR OPERATION

### 4.0 SURVEILLANCE REQUIREMENTS

3. Functional testing of snubbers shall be conducted at least once each operating cycle during cold shutdown. Ten percent of the total number of each brand of snubber shall be functionally tested either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria in Specification 4.6.H.4 below, an additional ten percent of that brand shall be functionally tested until no more failures are found or all snubbers of that brand have been tested.

The representative sample selected for functional testing shall include the various configurations, operating environments, and the range of size and capacity of the snubbers.

In addition to the regular sample and specified re-samples, snubbers which failed the previous functional test shall be retested during the next test period if they were reinstalled as a safety-related snubber. If a spare snubber has been installed in place of a failed safety related snubber, it shall be tested during the next period.

If any snubber selected for functional testing either fails to lockup or fails to move (i.e. frozen in place) the cause shall be evaluated and if caused by manufacturer or design deficiency, all snubbers of the same design subject to the same defect shall be functionally tested.

### 3.0 LIMITING CONDITIONS FOR OPERATION

- d. During reactor isolation conditions the reactor pressure vessel shall be depressurized to <200 psig at normal cooldown rates if the suppression pool temperature exceeds 120°F.
- e. The suppression chamber water volume shall be  $\geq 68,000$  and  $\leq 72,910$  cubic feet.
- f. Two channels of torus water level instrumentation shall be operable. From and after the date that one channel is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding 30 days unless such channel is sooner made operable. If both channels are made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding six hours unless at least one channel is sooner made operable.

#### 2. Primary Containment Integrity

Primary containment integrity, as defined in Section 1, shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212°F and fuel is in the reactor vessel except while performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed 5 Mw(t).

### 4.0 SURVEILLANCE REQUIREMENTS

- d. Whenever there is indication of relief valve operation with a suppression pool temperature  $\geq 160^\circ\text{F}$  and the primary coolant system pressure >200 psig, an extended visual examination of the suppression chamber shall be conducted before resuming power operation.
  - e. The suppression chamber water volume shall be checked once per day.
  - f. The suppression chamber water volume indicators shall be calibrated semi-annually.
- #### 2. Primary Containment Integrity
- a. Integrated Primary Containment Leak Test (IPCLT)

The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972:

- 1. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at  $40 \pm 10$  month intervals\* during shutdown at  $\bar{P}$  (41 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

\*The second test of the second 10-year service period may be conducted during the 1989 refueling outage.