REGULATO[redacted] INFORMATION DISTRIBUTION [redacted]STEM (RIDS)

ACCESSION NBR: 8607230171 DOC. DATE: 86/06/30 NOTARIZED: NO DOCKET #
FACIL: 50-263 Monticello Nuclear Generating Plant, Northern States 05000263
AUTH. NAME AUTHOR AFFILIATION
MUSOLF, D. Northern States Power Co.
RECIP. NAME RECIPIENT AFFILIATION
Office of Nuclear Reactor Regulation, Director (post 851125

SUBJECT: Forwards Rev 4 to updated SAR for Monticello Nuclear Generating Plant, including apps to FSAR, description & summary of safety evaluation changes, tests & experiments under 10CFR50.59 & updated SAR page changes.

Revised 7/28/86 DRB

DISTRIBUTION CODE: A053D COPIES RECEIVED: LTR 1 ENCL 13 ON SHELF SIZE: 3+1,400
TITLE: OR Submittal: Updated FSAR (50.71)

NOTES:

	RECIPIENT ID CODE/NAME		COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME		COPIES LTTR	COPIES ENCL
	BWR ADTS		1	0	BWR EB		1	0
	BWR EICSB		2	0	BWR FOB		1	0
	BWR PD1 LA		1	0	BWR PD1 PD		1	0
	AULUCK, R	01	1	1	BWR PSB		1	0
	BWR RSB		1	0				
INTERNAL:	IE FILE	05	1	1	NRR ROE, M. L		1	1
	REG FILE	02	1	1	RGN3	06	2	2
EXTERNAL:	DMB/DSS		1	1	FRANKLIN		1	1
	LPDR	04	1	1	NRC PDR	03	1	1
	NSIC		1	1				

TOTAL NUMBER OF COPIES REQUIRED: LTTR 20 ENCL 11

Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

June 30, 1986

Director
Office of Nuclear Reactor Regulation
US Nuclear Regulatory Commission
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Submittal of Revision No. 4 to the Updated Safety Analysis Report (USAR)

Pursuant to 10 CFR 50.71(e) we are submitting 13 copies of Revision No. 4 to the Updated Safety Analysis Report (USAR) for the Monticello Nuclear Generating Plant. This revision updates the information in the USAR for the period from January 1, 1985 through December 31, 1985.

Exhibit A contains a description and summary of the safety evaluation for changes, tests and experiments made under the provisions of 10 CFR 50.59 during this period.

Exhibit B contains the USAR page changes and instructions for entering the pages.

This revision also includes a reprinting of the appendices of the original Final Safety Analysis Report. Three new volumes are provided for this purpose.

There have been no changes to the Northern States Power Company Operational Quality Assurance Plan, the Licensed Operator Training Program, or the Licensed Operator Requalification Program. The Training Programs and the Quality Assurance Plan have been relocated from the Chapter 13 appendices to Appendix B and Appendix C, respectively.

David Musolf
Manager - Nuclear Support Services

c: Regional Administrator-III, NRC
NRR Project Manager (w/o Exhibit B), NRC
NRR Backup Project Manager (w/o Exhibit B), NRC
G Charnoff (w/o Exhibit B)

Attachments

A053

1/13

Exhibit A

MONTICELLO NUCLEAR GENERATING PLANT

ANNUAL REPORT OF CHANGES, TESTS AND EXPERIMENTS

The following sections include a brief description and a summary of the safety evaluation for those changes, tests and experiments which were carried out without prior NRC approval, pursuant to the requirements of 10CRF50.59(b).

1. SRI # 130 III, Downgrade Seismic Classification of Gaseous Radwaste

Description of Design Change:
The seismic design criteria for portions of the gaseous radwaste system was downgraded to Seismic Category II. The only portion of the offgas system required to meet the seismic design requirements is that portion of the system which includes the storage tanks and the attached piping up to the first isolation valve.

Summary of Safety Evaluation:
The seismic criteria for the gaseous radwaste system meets recommendations of Regulatory Guide 1.143.

2. SRI # 85-04, Pressure Gauges on RHRSW Header Vent Taps

Description of Design Change:
Two pressure gauges were added to the Residual Heat Removal Service Water system header vents.

Summary of Safety Evaluation:
The gauges were included in the seismic analysis of the service water lines. The stress on the lines, with the gauges included, were below the allowable stresses.

3. SRI # 85-05, Fuse Rating Type Discrepancies

Description of Design Change:
As a result of a breaker and fuse coordination study, a field walkdown of fuses in distribution panels showed that a number of discrepancies existed between the controlled drawings and as-found conditions. The discrepancies involved both fuse type and fuse ratings.

Summary of Safety Evaluation
There were no fuses replaced as a result of this review. The discrepancies in fuse type involved the use of time delay rather than non-time delay fuses. This, however, would not have resulted in an overload of the branch circuit conductors or resulted in a compromise of the short circuit interrupting capability. Discrepancies of fuse ratings, both smaller and greater ratings, were evaluated and found not to compromise the overcurrent protection required for the branch circuit conductors.

4. SRI # 85-07, Removal Without Replacement of One of Two Redundant CRD Pump Suction Relief Valves

Description of Design Change:
A redundant relief valve was removed on the suction line to #12 CRD pump. The other valve was renumbered to prevent confusion regarding relief valve identification. This eliminates unnecessary maintenance on the redundant relief valve.

Summary of Safety Evaluation:
The removal of the redundant relief valve does not adversely affect the safety or operability of any safety related equipment or components. The new configuration meets all original system overpressure protection and design requirements.

5. DC 78-020, Modification of Hot Machine Shop Exhaust System Ventilation

Description of Design Change:
The hot machine shop ventillation exhaust was redirected to provide radiation monitoring of the exhaust air. Previously this air was exhausted directly to the atmosphere.

Summary of Safety Evaluation:
Redircting the hot machine shop exhaust to the reactor plenum, which is monitored, satisfies the requirement that all releases that have a potential for being contaminated are monitored.

6. DC 83-087, Fireproofing of Structural Steel

Description of Design Change:
Section III.G.2 of Appendix R to 10CFR50 requires structural steel forming a part of or supporting a rated fire barrier to be protected to an equivalent rating. Barriers separating components of redundant minimum safe shutdown systems have, therefore, had their associated structural steel members coated with a cementitious fireproofing material to achieve a 3-hr fire resistance rating.

Summary of Safety Evaluation:
This modification provides increased fire protection for safe shutdown equipment. Non-asbestos fireproofing has been applied to structural steel. It is judged light enough as not to degrade the structural integrity of the existing steel.

7. MOD 85-012, Reroute of V-EF-9 Discharge Ductwork

Description of Design Change:
Reroute of the exhaust duct V-EF-9 provides a monitored release rather than a release directly to outside the air.

Summary of Safety Evaluation:
With a fire in the 4 kV bus room, smoke will now be exhausted to the turbine operating floor and removed there by the air handling units and directed to the reactor building plenum. Turbine building dP has not been increased by this modification. The unmonitored radiation release point that previously existed has been eliminated.