

AEC DISTRIBUTION FOR PART 50 DOCKET MATERIAL
(TEMPORARY FORM)

CONTROL NO: 3115

FROM: Northern States Power Co. Minneapolis, Minn. 55401 L.O. Mayer	DATE OF DOC: 6-2-72	DATE REC'D 6-7-72	LTR X	MEMO	RPT	OTHER
TO: Mr. A. Giambusso	ORIG 1 signed	CC 39	OTHER	SENT AEC PDR ✓ SENT LOCAL PDR ✓		
CLASS: U PROP INFO	INPUT	NO CYS REC'D 40	DOCKET NO: 50-263			

DESCRIPTION: Ltr submitted as rpt on the 2-26-72 problem when all four stop valves tripped closed simultaneously causing a reactor scram/ attach Chart & Locations of safety/relief valve....

ENCLOSURES:

**DO NOT REMOVE
ACKNOWLEDGED**

PLANT NAMES:

FOR ACTION/INFORMATION DL 6-8-72

BUTLER(L) W/ Copies	KNIEL(L) W/ Copies	VASSALLO(L) W/ Copies	ZIEMANN(L) W/ 9 Copies	KNIGHTON(ENVIRO) W/ Copies
CLARK(L) W/ Copies	SCHWENCER(L) W/ Copies	H. DENTON W/ Copies	CHITWOOD(FM) W/ Copies	W/ Copies
GOLLER(L) W/ Copies	STOLZ(L) W/ Copies	SCHEMEL(L) W/ Copies	DICKER(ENVIRO) W/ Copies	W/ Copies

INTERNAL DISTRIBUTION

REG FILES AEC PDR	STELLO-L	VOLLMER-L	KARAS-L L/A PWR
REG OPER (2)	MOORE-L	DENTON-L (4)	MASON-L L/A BWR
CC-RM P-506	LANGE-L	GRIMES-L	BROWN-L L/A PWR
MUNTZING & STAFF	PAWLICKI-L	GAMMILL-L	WILSON-L L/A PWR
GIAMBUSSO-L	THOMPSON-L	KNIGHTON-ENVIRO	KARI-L L/A BWR
BOYD-L-BWR	TEDESCO-L	DICKER-ENVIRO	SMITH-L L/A BWR
DEYOUNG-L-PWR	LONG-L	PROJ LDR ENVIRO:	GEARIN-L L/A BWR
MULLER-L-ENVIRO	MAINAS-L	SALTZMAN-IND.	DIGGS-L L/A
SKOVHOLT-L-OPER	SHAO-L	McDONALD-PLANS	TEETS-L L/A
KNUTH-L	BENAROYA-L	NUSSBAUMER-FM	WADE-L L/A ENVIRO
MACCARY-L	MORRIS-RO	SMILEY-FM	BRAITMAN-A/T
SCHROEDER-L	DUBE-L	P. COLLINS-L	HARLESS-ENVIRO
	E. CASE-L		

EXTERNAL DISTRIBUTION

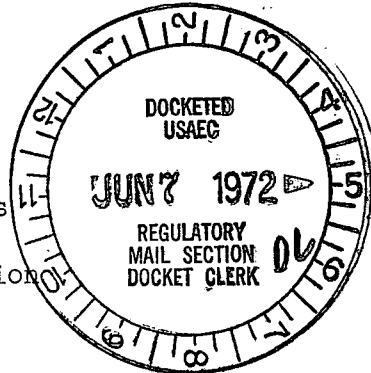
1-LOCAL PDR <u>Minneapolis, Minn.</u>	9-NATIONAL LAB'S ANL/ORNL/BNWL	1-SAN/LA/NY--PDR
1-DTIE-(LAUGHLIN)	1-R. CARROLL-OC, GT	1-CHIEF WATER REACTORS
1-NSIC-(BUCHANAN)	1-R. CATLIN, A-170, GT	1-RD....E. HALL F-309 GT
1-ASLB-YORE/SARYE WOODWARD/H. ST.	1-CONSULTANT'S NEWMARK/BLUME/AGBABIAN	
1-C. MILES-C-459, GT	1-DR. GERALD S. LELLOUCHE	
16 CYS ACRS-HOLDING	BROOKHAVEN NATIONAL LAB	

NSP**NORTHERN STATES POWER COMPANY**

MINNEAPOLIS, MINNESOTA 55401

June 2, 1972

Mr. A Giambusso
 Deputy Director for Reactor Projects
 Directorate of Licensing
 United States Atomic Energy Commission
 Washington, D C 20545



Dear Mr. Giambusso:

MONTICELLO NUCLEAR GENERATING PLANT
 Docket No. 50-263 License No. DPR-22

Momentary Opening of "A" Reactor Safety Valve

This letter is being submitted for your information on the subject momentary opening.

At 0530 hours on February 26, 1972, while performing the turbine stop valve 10% closure surveillance test, all four stop valves tripped closed simultaneously causing a reactor scram. Actuation of the safety/relief valves limited the maximum transient reactor pressure to a recorded value of 1118 psia. Incident to this event was a 0.9 psia increase in drywell pressure (see attachment) and a 100°F increase in "A" safety valve discharge nozzle temperature. Mid-level drywell temperature was 93°F prior to the scram and an approximate 6°F temperature rise was recorded at mid-level following the scram. Following this indication of a momentary actuation of the "A" safety valve, it reseated with no detectable subsequent leakage. There was no evidence of instrument or equipment malfunction or damage due to the momentary actuation.

The stop valve closure resulted from a momentary loss of turbine emergency trip oil pressure, which was caused by a malfunction of the test-exercise controls for the No. 2 stop valve. Because of the unusual manner in which the trip oil pressure was momentarily lost, the turbine control valves were not automatically tripped closed as they would be for a normal turbine trip. The pressure control system responded to the steam pressure transient by fully opening the turbine control valves to a position equivalent to 110% of rated steam flow. However, no steam flow could take place through the control valves since the stop valves, located immediately upstream, were already closed. The control system is designed to limit opening of turbine valves to prohibit steam flows greater than 110% of rated, therefore the bypass valves were prevented from opening and the pressure transient was more severe than for a normal turbine trip.

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LB

Mr. A Giambusso

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June 2, 1972

On Saturday, May 13, 1972, during an outage which included scheduled maintenance work in the drywell area, an inspection of the drywell equipment was performed and this inspection confirmed the indicated momentary actuation of the "A" safety valve. The aluminum cover on a blank flange downstream of "A" safety valve had dislocated. The surrounding surfaces had insulation deposits resulting from steam impingement on the flange insulation. An inspection in the reciprocal direction of the tee discharge disclosed the dislocation of a cable tray cover.

The nominal setting of "A" safety valve is 1210 psig. The respective locations of the safety/relief valve (RV2-71A) and the safety valve (RV2-70A) on the "A" main steam line are indicated in the second attachment.

Yours truly,



L O Mayer, P.E.
Director-Nuclear Support Services

LOM/br

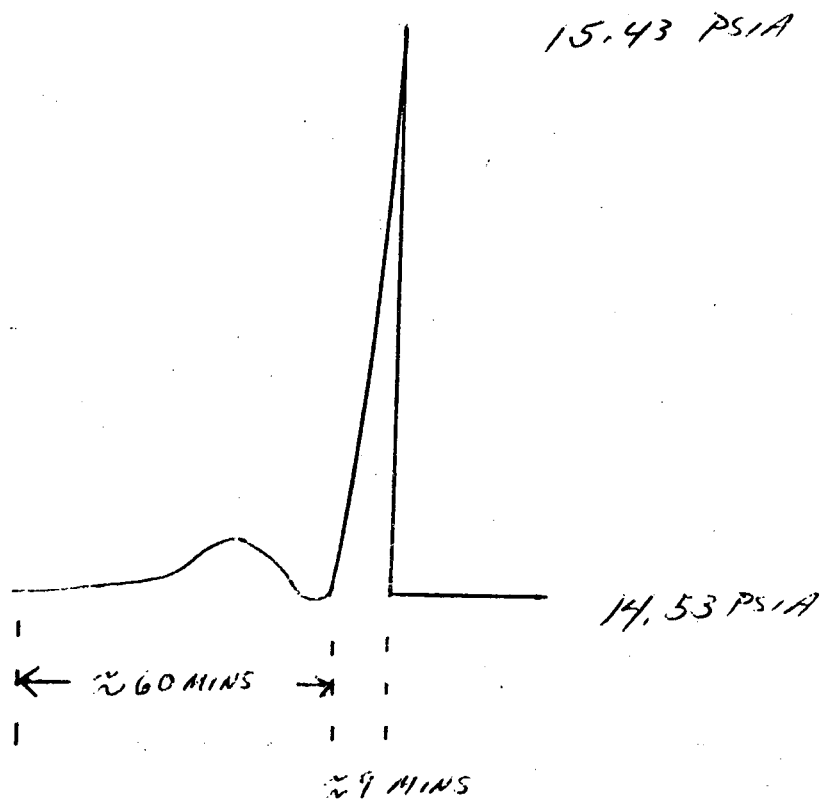
cc: B H Grier

Attachment

Regulatory

File Cy.

Received w/lt dated 6-2-72



DRYWELL PRESSURE
RECORDER PR 2994
CHART SPEED 1/2" - HOUR
SCALE 13 - 17 PSIA

REACTOR NOZZLE- N-3A

1'-0 1/16" 10'-8 9/16"

6-2-72

PSI-18-ED-1
FI-48

6"-1500# BLIND FLANGE
LARGE TONGUE

6"-1500# W/N FLC
LG. GROOVE

CONTINUED DWG AGAZI EL 353-0
RV24-10"-HE

FLOW NOZZLE SECT-12-10 LG.
(FURN. BY OTHERS)
INCLUDES 4" PORT-FIT-UP

REACTOR
IGN

NOT
COMPLETE

NOT
COMPLETE

NOT
COMPLETE

NOT
COMPLETE

NOT
COMPLETE

PSI-18-ED-3
FI-52

