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OGC, ROOM P-506A	SCHROEDER	GAMMILL	✓DIGGS (L)	SALTZMAN
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P. COLLINS	ROSS	KNIGHTON	TEETS (L)	
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✓REG OPR	TEDESCO	REGAN	WILLIAMS (E)	
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1 - LOCAL PDR Minneapolis, Minn.

1 - DTIE (ABERNATHY)

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WOODARD/"H" ST. 16 - CYS ACRS HOXXXXXX SENT TO LIC. ASST.

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RM-B-127, GT.

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NORTHERN

MINNEAPOLIS, MINNESOTA 55401

November 30, 1973

Mr. D. J. Skovholt Assistant Director for Operating Reactors Directorate of Licensing United States Atomic Energy Commission

Washington, DC 20**5**45





Dear Mr. Skovholt:

MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

Hydraulic Shock Suppressors and Restraints

Re-inspection of hydraulic shock suppressors at the Monticello Nuclear Generating Plant was completed during a scheduled outage November 14 through November 17, 1973. Inspection results, corrective measures, and other information requested in your letter to Mr. L O Mayer, dated October 1, 1973, are contained in this report.

All shock suppressor units located outside primary containment have been inspected biweekly since our October 9, 1973 report. All units within the primary containment were reinspected November 14, 1973. There inspections identified no indications of inoperability or loss of oil from any of the units.

As of our October 9, 1973 report all units within the primary containment (drywell) had been rebuilt using vendor supplied materials which included seal materials other than ethylene propylene. During this November 14-17, 1973 outage, all units within the drywell were again rebuilt to provide exclusively ethylene propylene for all internal soft parts. Additionally the oil-fill fittings were replaced with high-temperature, high pressure leakproof fittings having Buna-N seats. Internal soft parts from the units rebuilt this outage were inspected and found to be free of any indication of deterioration. Parts unknown to be of Buna-N material experienced no deterioration in the several months service since installation and in this regard the replacement oil-fill fittings are believed to be compatible with the service of this application.

Several units within the drywell were previously equipped with TLD's and the radiation dosage determined. Additionally, four units were recently provided with thermocouples and nine others with temperature indicating tape to provide a measure of the temperature environment. Initial values of these parameters are included in the attached Table I - Radiation and Temperature Measurements for Hydraulic Shock Suppressors.

The corrective actions implemented this outage in conjunction with the results of surveillance inspections to date are believed to provide a reasonable basis for continued operation to the 1974 spring refueling outage.

The surveillance inspection of units located outside the primary containment will be continued on a monthly basis and all shock suppressors within the drywell will be inspected during the 1974 spring refueling outage. During this interim period of operation those units within the drywell will be inspected only during scheduled or unscheduled outages for which drywell entry may be required for other purposes. Additionally, those units outside the drywell will be rebuilt on an individual basis, as feasible, to provide ethylene propylene internal parts. It is anticipated that all units outside the drywell will have been rebuilt by the conclusion of the refueling outage.

Yours very truly,

L O Mayer, PE

Director of Nuclear Support Services

LOM/1h

cc: J G Keppler

G Charnoff

Minnesota Pollution Control Agency

Attn. K Dzugan

L.O. Mayer

 $\underline{\text{TABLE I}} \ - \ \underline{\text{Radiation and Temperature Measurements for Hydraulic Shock Suppressors}}$

UNIT	LOCATION	RADIATION DOSE (REM) FOR A FORTY-NINE DAY PERIOD
SS-1	DRYWELL MAINSTEAM 953	6,234
SS-4BR	DRYWELL RECIRC 934'	40,471
SS-5AR	DRYWELL RECIRC 941'	2,028
UNIT	LOCATION	TEMPERATURE (°F) AT 95 PERCENT POWER
SS-1	DRYWELL MAINSTEAM 953'	109
SS-1BR	DRYWELL RECIRC 922'	104
SS-3	DRYWELL MAINSTEAM 950'	118
SS-20	DRYWELL RHR 964'	98