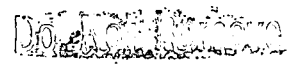
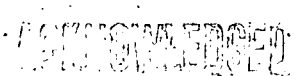


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CONTROL NO: 6005

FILE: _____

FROM: Northern States Power Co. Minneapolis, Minn. 55401 L.O. Mayer			DATE OF DOC 5-30-75	DATE REC'D 6-2-75	LTR XX	TWX	RPT	OTHER
TO: Mr. D.L. Ziemann			ORIG 1 signed	CC 39	OTHER	SENT AEC PDR XX SENT LOCAL PDR XX		
CLASS	UNCLASS XXX	PROP INFO	INPUT	NO CYS REC'D 40		DOCKET NO: 50-263		
DESCRIPTION: Ltr re our 4-17-75 ltr...furn addl info re containment design info & furn schedule of response to info re containment...				ENCLOSURES:				
PLANT NAME: Monticello				<div style="text-align: right;">   </div>				

FOR ACTION/INFORMATION

DHL 6-3-75

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✓ 1 - Newton Anderson		1 - J. D. RUNKLES, Rm E-201
✓ 14 ACRS HOLDING/SENT		GT

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Regulatory Docket File

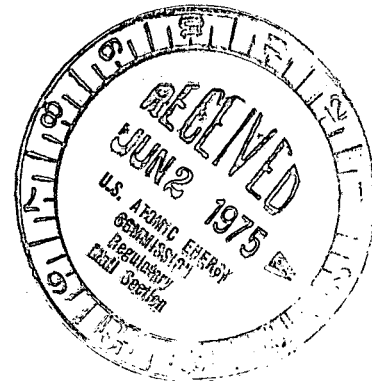
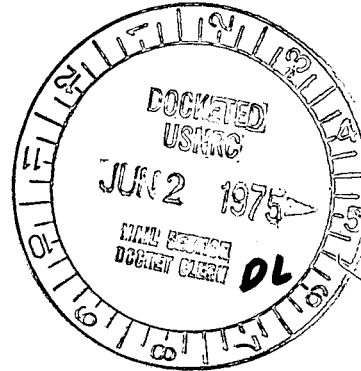
NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

May 30, 1975

Mr D L Ziemann, Chief
Operating Reactors Branch #2
Division of Reactor Licensing
U S Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr Ziemann:



MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Additional Containment Design Information

As requested by your April 17, 1975, letter, an evaluation of the Monticello containment configuration details will be made and documented, together with a plant evaluation similar to that performed by GE and discussed with the NRC in a meeting in Bethesda on April 10, 1975. It is expected that this will be submitted by August 4, 1975. This evaluation will address the short term concerns on the effect of pool swell on the Drywell-To-Torus vent system. It will include large size plan and section drawings of the suppression chamber which illustrate the structures, equipment, and piping in the suppression pool which could be subjected to hydrodynamic loads.

Your letter identified several other specific items of information that will require an expansion of the above evaluation. A tentative program and schedule of response to these questions are outlined below:

Document S/R Valve Vent Clearing Model	July 31, 1975
Impact Tests for Torus Internal Structure Shapes	July 31, 1975
Qualify Impact Model	September 30, 1975
Calculate Mark I Pool Swell Impact Load for the Reference Plant	September 30, 1975
Determine Need for Plant Torus Stress Test for S/R Valve Discharge	September 30, 1975
Calculate other Significant Mark I LOCA Dynamic Loads	December 31, 1975
Complete Currently Planned Mark I Pool Swell Tests	December 31, 1975

6005

NORTHERN STATES POWER COMPANY

Mr D L Ziemann

Page 2

May 30, 1975

Confirm Calculated Loads

March 31, 1976

Perform Stress Analysis/Test of Torus, Torus
Internals and Supports

September 30, 1976

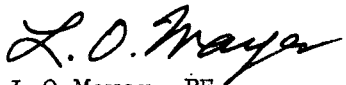
Combine Stresses and Compare to Plant Licensing
Bases

December 31, 1976

In accordance with your February 14, 1975, Safety/Relief Valve letter requesting a program identification, we should be able to submit details on the above tentative program by July 7, 1975. Our program will be designed to respond to both your February 14, 1975, letter and your recent April 17, 1975, letter and will consider both the S/R valve and LOCA conditions for a time history of potential loads including assymetric loads and a justification of these loads.

Subsequent to the submittal of our program identification, we will meet with you as part of the BWR owners group that has been formed to address this common problem.

Yours very truly,



L O Mayer, PE
Manager of Nuclear Support Services

LOM/LLT/ak

cc: J G Keppler
G Charnoff
Minnesota Pollution Control Agency
Attn: E A Pryzina