



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION III  
799 ROOSEVELT ROAD  
GLEN ELLYN, ILLINOIS 60137

SEP 28 1977

Docket No. 50-263 *177-16*

Northern States Power Company  
ATTN: Leo Wachter  
Vice President  
Power Production and  
System Operation  
414 Nicollet Mall  
Minneapolis, MN 54401

Gentlemen:

This refers to the inspection conducted by Mr. J. L. Barker of this office on September 12-16, 1977, of activities at Monticello Nuclear Generating Plant authorized by NRC Operating License No. DPR-22 and to the discussion of our findings with Mr. L. Eliason and others of your staff at the conclusion of the inspection.

The enclosed copy of our inspection report identifies areas examined during the inspection. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations, and interviews with personnel.

No items of noncompliance with NRC requirements were identified during the course of this inspection.

In accordance with Section 2.790 of the NRC's "Rules of Practice," Part 2, Title 10, Code of Federal Regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC's Public Document Room, except as follows. If this report contains information that you or your contractors believe to be proprietary, you must apply in writing to this office, within twenty days of your receipt of this letter, to withhold such information from public disclosure. The application must include a full statement of the reasons for which the information is considered proprietary, and should be prepared so that proprietary information identified in the application is contained in an enclosure to the application.



Northern States Power  
Company

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We will gladly discuss any questions you have concerning this inspection.

Sincerely,

Caston Fiorelli, Chief  
Reactor Operations and  
Nuclear Support Branch

Enclosure: IE Inspection  
Report No. 50-263/77-16

cc w/encl:  
Mr. L. R. Eliason,  
Plant Manager  
Central Files  
Reproduction Unit NRC 20b  
PDR  
Local FDR.  
NSIC  
TIC  
Anthony Moisman, Esq.,  
Attorney

OFFICE	R111	R111	R111	R10R111		
SURNAME	Barker/ls	Little	Fiorelli	for Choules		
DATE	9/27/77			9/27/77		

U.S. NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-263/77-16

Docket No. 50-263 License No. DPR-22

Licensee: Northern States Power Company  
414 Nicollet Mall  
Minneapolis, MN 55401

Facility Name: Monticello Nuclear Generating Plant

Inspection at: Monticello Site, Monticello, MN

Inspection Conducted: September 12-16, 1977

Inspector: *J. L. Barker*  
J. L. Barker

9/27/77

Approved by: *W. S. Little*  
W. S. Little, Chief  
Nuclear Support Section

9/27/77

Inspection Summary

Inspection on September 12-16, 1977 (Report No. 50-263/77-16)

Areas Inspected: Routine, unannounced inspection of main steam line isolation valve local leakrate testing, preparation for a refueling outage, pre-fuel handling activities, fuel handling activities, maintenance during a refueling outage, and preparation for startup following a refueling outage. The inspection involved 30 inspector-hours onsite by one NRC inspector and included inspection effort conducted during offshift hours.  
Results: No items of noncompliance or deviations were identified.

## DETAILS

### 1. Persons Contacted

- \*L. Eliason, Plant Manager
- \*W. Anderson, Superintendent, Operations and Maintenance
- \*M. Clarity, Superintendent, Plant Engineering and Radiation Protection
  - D. Antony, Plant Engineer, Operations
  - W. Shamla, Plant Engineer, Technical
- \*R. Scheinost, Quality Engineer
  - S. Pearson, Operations Supervisor
  - D. Nevinski, Engineer Nuclear
  - R. Van Dell, Engineer
  - D. Wegener, Engineer

The inspector also contacted eleven other licensee employees. They included three shift supervisors, two lead plant equipment operators, and six plant equipment operators.

\*Denotes those present at the exit interview.

### 2. Main Steam Line Isolation Valve (MSIV) Local Leak Rate Testing

The inspector reviewed licensee procedure CCD 0137-7A, "Reactor Steam Supply Valve Leak Rate Testing by Pressurizing the Reactor with Air" and Test No. 0137, "Local Leak Rate Test." He reviewed the results of MSIV local leak rate test and verified adherence to approved procedures and proper acceptance criteria. All MSIV's except AO-2-86A and AO-2-80A, which will be retested after proper repairs during the refueling outage, received satisfactory results.

No items of noncompliance or deviations were identified.

### 3. Preparation for a Refueling Outage

The inspector verified that approved procedures are available for new fuel receipt and inspection, fuel transfer and core verification, recycled fuel inspections, and fuel sipping operations; and that new fuel was received and inspected in accordance with the licensee's procedures.

The inspector reviewed licensee procedures No. 9001, "Reactor Well and Dryer-Separator Storage Pool Filling Procedure," No. 9006, "Reactor Well and Dryer-Separator Storage Pool Draining Procedure," No. 9007B, "Shift Supervisor's Refueling Checklist," No. 9008,

"In-Core Sipping Procedure," No. 9008A, "Out-of-Core Sipping Procedure," No. 9012, "Fuel Receiving Procedure," No. 9015, "Inspection of New Fuel and Channels Followed by Placement Into the Vault, Placement Into the Pool, or Transfer from the Vault to the Pool," No. 9023, "Refueling Bridge Load Cell Calibration," No. 9024, "Core Reload Verification Procedure," No. 9026 "Refueling Bridge Functional Test," and Operations Manual Sections C.3, "Shutdown Procedures," D.1., "Accountability" and D.4., "Refueling Outage Procedures." The inspector noted that a great deal of beneficial preplanning had been accomplished by the licensee which tended to make the refueling outage follow in a continuous manner.

No items of noncompliance or deviations were identified.

4. Pre-Fuel Handling Activities

The inspector verified that surveillance testing had been completed on Technical Specification requirements, refueling machine operation and indexing, ventilation requirement in fuel storage areas, refueling interlocks, crane testing, refueling deck radiation monitors, and communication systems.

The inspector reviewed the Shift Supervisor's Log and the Control Room Operator's Daily Log for the dates September 9-11, 1977. He also reviewed licensee surveillance tests No. 0201/0202, "Refueling Interlocks Weekly Tests," No. 2164, "Plant Pre-Shutdown Checklist," No. 9007B, "Shift Supervisor's Refueling Checklist," and No. 9026, "Refueling Bridge Functional Test."

No items of noncompliance or deviations were identified.

5. Fuel Handling Activities

The inspector verified by direct observations that core monitoring during refueling operations was in accordance with Technical Specifications, that fuel bundle insertion and removal were in accordance with established procedures, that core internals were stored to protect against damage, that housekeeping was proper, that vessel level was in accordance with Technical Specifications, that the mode switch was as required by Technical Specifications, that control blade checks were in accordance with approved procedures, that the individual directing fuel handling activities held a senior operator license and was present directly supervising activities, and that a licensed reactor operator was present in the control room and in constant direct communications with a member of the fuel handling crew when work was being performed that could affect the reactivity of the reactor.

The inspector noted that all activities conducted by operations personnel both in the control room and on the refueling bridge were conducted in a professional manner with core safety as the primary objective.

No items of noncompliance or deviations were identified.

6. Maintenance During a Refueling Outage

The inspector verified that maintenance activities conducted during the refueling outage were being conducted by qualified personnel in accordance with approved procedures. He verified the procedures written to control work activities contained administrative approvals for removing a system from service and returning it to service, hold points for inspection and signoff by quality assurance personnel, provisions for testing following maintenance, provisions for jumper removals, and responsibility for reporting to licensee management details concerning design or construction related deficiencies identified during maintenance.

The inspector reviewed licensee procedure 4-ACD-3.6, "Work Request Authorization." He also witnessed the maintenance and calibration activities conducted on the Target Rock Safety/Relief Valves.

No items of noncompliance or deviations were identified.

7. Preparations for Startup Following a Refueling Outage

The inspector verified that systems that were disturbed or tested during the refueling outage will be returned to an operating status prior to plant startup and that plant startup procedures require adherence to the licensee's Technical Specifications and commitments, as they pertain to startup testing and power operation prerequisites.

The inspector reviewed licensee procedures No. 2100, "Master Systems Checklist," No. 2150, "Pre-Startup Master Checklist," No. 2167, "Startup Checklist," and Operation Manual Section C.1., "Startup Procedures."

No items of noncompliance or deviations were identified.

8. Exit Interview

The inspector met with licensee representatives (denoted in Paragraph 1) at the conclusion of the inspection on September 16, 1977. The inspector summarized the scope and findings of the inspection.