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ACCESSION NBR: 8003240564      DOC. DATE: 80/03/21      NOTARIZED: NO      DOCKET #  
 FACIL: 50-263 Monticello Nuclear Generating Plant, Northern States      05000263  
 AUTH. NAME      AUTHOR AFFILIATION  
 MYRABO, A. L.      Northern States Power Co.  
 RECIP. NAME      RECIPIENT AFFILIATION  
 MYRABO, A. L.      Region 3, Chicago, Office of the Director

SUBJECT: LER 80-013/01T-0: on 800309, during refueling operations, rod withdrawn block not received when refueling bridge traversed core. Caused by actuating arm out of adjustment. Components readjusted & interlocks tested prior to refueling.

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# NSP

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

March 21, 1980

Mr J G Keppler  
Office of Inspection & Enforcement  
U S Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Dear Mr Keppler:

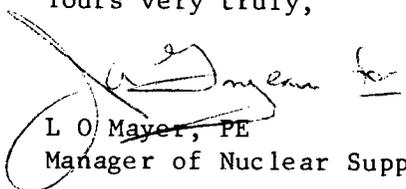
MONTICELLO NUCLEAR GENERATING PLANT  
Docket No. 50-263 License No. DPR-22

Refueling Rod Block Interlock Failure

The Licensee Event Report for this occurrence is reproduced on the back of this letter. Enclosed are three copies.

This event is reported in accordance with Technical Specification 6.7.B.1.g. in that the condition (limit switch out of adjustment) required refueling operations to be stopped in accordance with Technical Specification 3.10.A.

Yours very truly,

  
L O Mayer, PE  
Manager of Nuclear Support Services

LOM/JAG/ak

cc: Director, IE, USNRC (40)  
Director, MIPC, USNRC (3)  
MPCA  
Attn: J W Ferman

-over-

APR 2 5 11

S

8003240 551



LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

LICENSEE CODE: M N M N P 1; LICENSE NUMBER: 00-000000-00; LICENSE TYPE: 411111; CAT 56: 4

REPORT SOURCE: L; DOCKET NUMBER: 05000263; EVENT DATE: 030980; REPORT DATE: 032180

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

At 0605, it was discovered that a rod withdrawal block was not received when the refueling bridge traversed the core during refueling operations. Technical Specification 3.10.A requires the refueling interlocks to be operable during core alterations. Interlocks had successfully been tested at 0400 the previous day. Records for that period confirmed that no control rods had been moved while fuel was moved. The event had no effect on the public health or safety. One previous occurrence. AO-75-02.

SYSTEM CODE: FD; CAUSE CODE: B; CAUSE SUBCODE: A; COMPONENT CODE: INSTRU; COMP. SUBCODE: S; VALVE SUBCODE: Z; LER/RO REPORT NUMBER: 80; EVENT YEAR: 80; SEQUENTIAL REPORT NO.: 013; OCCURRENCE CODE: 01; REPORT TYPE: T; REVISION NO.: 0; ACTION TAKEN: EF; EFFECT ON PLANT: Z; SHUTDOWN METHOD: Z; HOURS: 0000; ATTACHMENT SUBMITTED: N; NPRD-4 FORM SUB.: N; PRIME COMP. SUPPLIER: N; COMPONENT MANUFACTURER: G080

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

Refueling operations were immediately stopped. Investigation revealed that the actuating arm of the limit switch (which opens when the bridge is over the core) was out of adjustment, preventing the switch from operating properly. The components were readjusted & interlocks tested prior to resuming refueling. Modifications of the components are being considered.

FACILITY STATUS: H; % POWER: 000; OTHER STATUS: NA; METHOD OF DISCOVERY: A; DISCOVERY DESCRIPTION: Operator observation

ACTIVITY CONTENT: Z; RELEASED OF RELEASE: Z; AMOUNT OF ACTIVITY: NA; LOCATION OF RELEASE: NA

PERSONNEL EXPOSURES NUMBER: 000; TYPE: Z; DESCRIPTION: NA

PERSONNEL INJURIES NUMBER: 000; DESCRIPTION: NA

LOSS OF OR DAMAGE TO FACILITY TYPE: Z; DESCRIPTION: NA

PUBLICITY ISSUED: N; DESCRIPTION: NA

GPO 917-926