

AEC DISTRIBUTION FOR PART 50 DOCKET MATERIAL
(TEMPORARY FORM)

CONTROL NO: 3677

FILE: 210

FROM: Northern States Power Company Minneapolis, Minnesota L. O. Mayer			DATE OF DOC 4-22-74	DATE REC'D 4-26-74	LTR X	MEMO	RPT	OTHER
TO: J. F. O'Leary			ORIG 1 signed	CC 39	OTHER	SENT AEC PDR <u>XXXX</u> SENT LOCAL PDR <u>XXXX</u>		
CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:			
	XXX			40	50-263			

DESCRIPTION:
Ltr furn info re abnormal occurrence discovered 4-12-74 in which two indications were found in the stainless steel tee associated with the flow switch

PLANT NAME: MONTICELLO

ENCLOSURES:

**DO NOT REMOVE
ACKNOWLEDGED**

FOR ACTION/INFORMATION 4-26-74 GMC

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| 1 - P. R. DAVIS (AEROJET NUCLEAR) | NEWMARK/BLUME/AGBABIAN | RM-B-127, GT. |
| <input checked="" type="checkbox"/> 16 - CYS ACRS HOLDING | 1-GERALD ULRIKSON...ORNL | 1-RD..MULLER..F-309 GT |
| Sent to Lic Asst Diggs 4-26-74 | 1-B & M SWINEBROAD, Rm E-201 GT | |

NSP

NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

April 22, 1974

Mr. J F O'Leary
Directorate of Licensing
Office of Regulation
U S Atomic Energy Commission
Washington, DC 20545

Dear Mr. O'Leary:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Cracked Tee in Standby Liquid Control System

A condition occurred at the Monticello Nuclear Generating Plant which we are reporting to your office in accordance with Section 6.7.B.1, Abnormal Occurrence Reports, of Appendix A, Technical Specifications, of Provisional Operating License DPR-22.

On April 12, 1974, while performing a non-destructive examination (Dye Penetrant test) of a socket weld on a Standby Liquid Control PEECO flow switch (following removal of the flow switch actuator paddle) two indications were found in the stainless steel tee associated with the flow switch.

The depth of the indications was explored and it was determined that they were non-injurious (as defined in the Specification for Wrought Austenitic Stainless Steel Piping Fittings, SA-403, in the ASME Boiler and Pressure Vessel Code, Section II). The indications were removed during the exploration. It is postulated that the indications were created in the manufacturing process.

No other indications have been detected in Standby Liquid Control System piping in the past and, because of the non-injurious nature of the indications discovered on April 12, no further inspection is considered necessary.

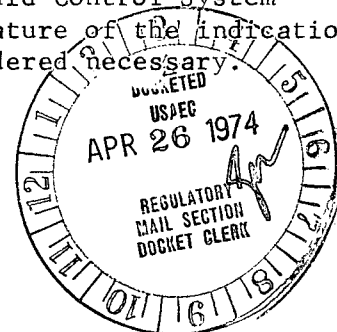
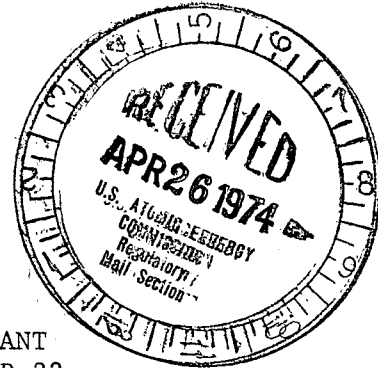
Very truly yours,

L. O. Mayer

L O Mayer, PE
Director of Nuclear Support Services

LOM/lh

cc: J G Keppler
G Charnoff
Minnesota Pollution Control Agency
Attn. E A Pryzina



3677

REGULATORY DOCKET FILE COPY

FROM: Northern States Power Company Minneapolis, Minnesota 55401 L. O. Mayer			DATE OF DOC 4-18-74	DATE REC'D 4-22-74	LTR X	MEMO	RPT	OTHER
TO: J. F. O'Leary			ORIG 1 signed	CC	OTHER	SENT AEC PDR X SENT LOCAL PDR X		
CLASS	UNCLASS XXX	PROP INFO	INPUT	NO CYS REC'D 40	DOCKET NO: 50-263			

DESCRIPTION:
Ltr reporting abnormal occurrence on 4-10-74, regarding a broken stem on RHR service water control valve, CV-1728.....

ENCLOSURES:

**DO NOT REMOVE
ACKNOWLEDGED**

PLANT NAME: Monticello

FOR ACTION/INFORMATION 4-22-74 GC

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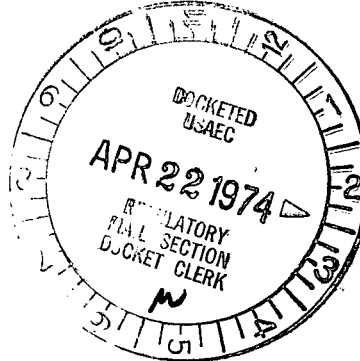
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| ✓GC, ROOM P-506A | SCHROEDER | GAMMILL | SALTZMAN |
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| 4-22-74 DIGGS | 1-B & M SWINEBROAD, Rm E-201 GT | |

April 18, 1974

Mr. J F O'Leary, Director
Directorate of Licensing
Office of Regulation
U S Atomic Energy Commission
Washington, DC 20545



Dear Mr. O'Leary:

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Broken Stem on RHR Service Water Control Valve, CV-1728

A condition occurred at the Monticello Nuclear Generating Plant which we are reporting to your office in accordance with Section 6.7.B.1, Abnormal Occurrence Reports, of Appendix A, Technical Specifications, of Provisional Operating License DPR-22.

On April 10, 1974, an inspection of the RHR Service Water Control Valve, CV-1728, revealed that the lower valve stem was broken. Further inspection revealed significant metal erosion below the lower port disc. (See attached figure) The nature of these failures and the design of the valve are such that the valve was still able to perform its function of controlling differential pressure between the RHR and RHR Service Water. The valve operator withdraws the stem to a given position and maintains that position through a position feedback device. The inner valve is designed to open when either normal flow or normal differential pressure forces are applied across the valve ports. Consequently, the inner valve will be forced open by hydraulic forces until it is restrained at the required position by the severed lower stem.

This is the first failure of this type which has been experienced. Metallurgical and mechanical investigations to determine the cause of the stem fracture and the erosion are continuing. The control valve in the redundant system will be inspected during the current outage. Following the completion of the metallurgical and mechanical tests and the inspection of the redundant system, a report on the final resolution of the problem will be submitted.

Yours very truly,

A handwritten signature in cursive script that reads "L. O. Mayer".

L O Mayer, PE
Director of Nuclear Support Services

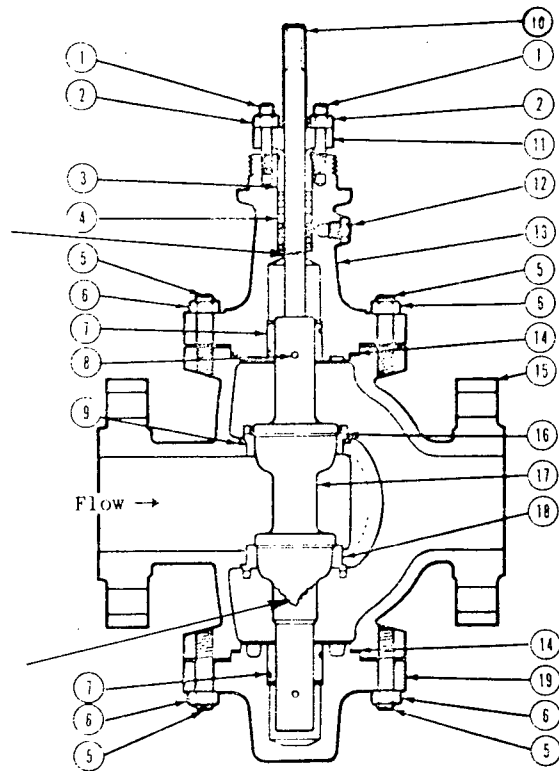
LOM/kn

3515

cc: J G Keppler
G Charnoff
Minnesota Pollution Control Agency
Attn. E A Pryzina

Fracture of
Lower Stem

Eroded Area



ITEM	DESCRIPTION
1	Stud, Hold Down
2	Nut, Hold Down
3	Packing Follower
4	Packing Assembly
5	Stud, Body
6	Nut, Body
7	Guide Bushing
9	Shim, Laminated
10	Lower Stem (Regular)
11	Hold Down Plate
12	Pipe Plug
13	Top Closure (Regular)
13a	Top Closure (Finned)
13b	Top Closure (Extended)
14	Body Gasket
15	Valve Body
16	Seat Ring
17	Inner Valve
18	Seat Ring
19	Bottom Closure
20	Grease Ring
21	Lubricator Assembly
22	Isolating Valve Assembly

□ Indicates Recommended Spare Parts

Figure 1. Typical Double Port Valve