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FILE: INCIDENT REPORT

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NORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

January 30, 1975

Mr A Giambusso, Director Division of Reactor Licensing U S Nuclear Regulatory Commission Washington, DC 20555



Dear Mr Giambusso:

MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

Licensee Event Report to the NRC Crack Indications in Recirculation Bypass Piping

Attached are the Licensee Event Report and a supplementary report for this occurrence.

Yours very truly,

L O Mayer, PE Manager of Nuclear Support Services

LOM/DMM/ak

cc: J G Keppler G Charnoff Minnesota Pollution Control Agency Attn: E A Pryzina



SUPPLEMENT TO LICENSEE EVENT REPORT TO THE NRC

Crack Indications in Recirculation Bypass Piping

Supplemental Information:

- 1. Report Number: A0 263/75-03
- 2a. Report Date: January 30, 1975
- 3. <u>Facility</u>: Monticello Nuclear Generating Plant Monticello, Minnesota 55362
- 4. Identification of Occurrence:

This report concerns the detection of crack-like indications in the Reactor Recirculation System 4-inch diameter bypass lines during the inservice inspection.

5. Conditions Prior to Occurrence:

Refueling Shutdown - A refueling outage was in progress at the time of the occurrence.

6. Description of Occurrence:

On January 20, 1975, during the inservice inspection of the reactor coolant system, linear indications were detected by ultrasonic examination in two welds in loop A and one weld in loop B of the reactor recirculation system 4-inch diameter bypass lines. Radiographic examination of these three welds definitely verified the presence of a linear indication in one of these welds. The radiograph showed the indication to be located in the weld heat-affected zone.

After a thorough review and study of all of the ultrasonic recordings and comparing inspection results obtained in November, 1974, with the present results, a fourth weld became a suspect. This weld was located in loop A. Although the amplitude of the ultrasonic signal for this weld was relatively low, the signal pattern was similar to the three welds that were rejected; therefore, this weld was also rejected (see attached Figure).

7. Designation of Apparent Cause of Occurrence:

The cause is unknown. It appears to be a generic problem similar to problems experienced at other BWR facilities. Identical inspections performed in November, 1974, revealed no unacceptable indications in these same welds. Indications which were detected in the November inspection were of a magnitude and nature such that they were indiscernible from the weld root I.D.

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EVENT DESCRIPTION [02] During inservice inspection linear indications were detected by ultrasonic	1
7 8 9 03 examination (two welds Loop A, one weld Loop B) of reactor recirculation 4-inc	
7 8 9 [04] humana lines. Pediesmenhs of the 2 wolds worified are indication (in best off	BO
7 B 9 [05] zone). No previous occurrence. Pipe to be replaced prior to startup (AO 75	$\frac{1}{1}$
7 8 9	BO
7 B 9 PRIME SYSTEM CAUSE COMPONENT COMPONENT	8 0
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Cause unknown. Laboratory analysis of removed piping to be performed. Piping	to
7 8 9 09 be reinspected during fall 1975 refueling outage.	SO
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7 8 9 FACILITY STATUS % POWER OTHER STATUS 11 H 0 0 NA D NA	80
7 8 9 10 12 13 44 45 46 FORM OF ACTIVITY CONTENT	SÕ
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16 Z NA 7 8 9 10	
7 8 9	BO
AODITIONAL FACTORS	
7 B 9	a was BO
19 conservatively rejected and removed.	
NAME: Phil Krumpos 612-330-6737	60

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8. Analysis of Occurrence:

This occurrence had no effect on the public health and safety. Assuming these linear indications are cracks, and assuming they would have propagated through the pipe wall, the resultant water leaks would have been detected by the drywell sump monitoring system. In the unlikely event of a gross failure of a four-inch bypass loop, the resultant break would be well within the capability of the ECCS System.

9. Corrective Action:

Segments of both pipe loops which contain the four rejected welds are being removed and replaced. This piping (schedule 80, type 304 stainless steel) will be sent to a laboratory for metallurgical examination.

10. Failure Date:

Not applicable.

