



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 28, 2011

Mr. Brian J. O'Grady
Vice President-Nuclear and CNO
Nebraska Public Power District
72676 648A Avenue
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - ISSUANCE OF AMENDMENT RE: NON-
CONSERVATIVE BATTERY TERMINAL VOLTAGE ON FLOAT CHARGE AND
SPECIFIC GRAVITY ACCEPTANCE CRITERIA IN TECHNICAL
SPECIFICATION (TAC NO. ME4974)

Dear Mr. O'Grady:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 239 to Renewed Facility Operating License No. DPR-46 for the Cooper Nuclear Station (CNS). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 29, 2010, as supplemented by letters dated June 10 and August 31, 2011.

The amendment revises the acceptance criteria in CNS TS 3.8.4, "DC [Direct Current] Sources - Operating," Surveillance Requirement (SR) 3.8.4.1, and TS 3.8.6, "Battery Cell Parameters," Table 3.8.6-1, "Battery Cell Parameter Requirements." Specifically, the amendment revises the acceptance criteria in TS SR 3.8.4.1 and TS Table 3.8.6-1 by revising the battery terminal voltage on float charge and specific gravity acceptance criteria to ensure that the safety-related batteries can perform their safety functions and will remain operable during postulated design basis events.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Lynnea E. Wilkins".

Lynnea E. Wilkins, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-298

Enclosures:

1. Amendment No. 239 to DPR-46
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 239
License No. DPR-46

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nebraska Public Power District (the licensee), dated October 29, 2010, as supplemented by letters dated June 10 and August 31, 2011, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

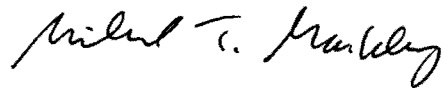
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-46 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A as revised through Amendment No. 239, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License No. DPR-46
and Technical Specifications

Date of Issuance: October 28, 2011

ATTACHMENT TO LICENSE AMENDMENT NO. 239

RENEWED FACILITY OPERATING LICENSE NO. DPR-46

DOCKET NO. 50-298

Replace the following pages of the Renewed Facility Operating License No. DPR-46 and Appendix A Technical Specifications with the enclosed revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating License

REMOVE

INSERT

-3-

-3-

Technical Specifications

REMOVE

INSERT

3.8-17

3.8-17

3.8-25

3.8-25

(5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2419 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 239, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Physical Protection

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Cooper Nuclear Station Safeguards Plan," submitted by letter dated May 17, 2006.

NPPD shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The NPPD CSP was approved by License Amendment No. 238.

(4) Fire Protection

The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Cooper Nuclear Station (CNS) Updated Safety Analysis Report and as approved in the Safety Evaluations dated November 29, 1977; May 23, 1979; November 21, 1980; April 29, 1983; April 16, 1984; June 1, 1984; January 3, 1985; August 21, 1985; April 10, 1986; September 9, 1986; November 7, 1988; February 3, 1989; August 15, 1995; and July 31, 1998, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.8.4.1	<p>Verify battery terminal voltage on float charge is:</p> <p>a. ≥ 125.9 V for the 125 V batteries; and</p> <p>b. ≥ 260.4 V for the 250 V batteries.</p>	7 days
SR 3.8.4.2	<p>Verify no visible corrosion at battery terminals and connectors.</p> <p><u>OR</u></p> <p>Verify battery connection resistance meets the limits specified in Table 3.8.4-1.</p>	92 days
SR 3.8.4.3	<p>Verify battery cells, cell plates, and racks show no visual indication of physical damage or abnormal deterioration that degrades battery performance.</p>	18 months
SR 3.8.4.4	<p>Remove visible corrosion and verify battery cell to cell and terminal connections are coated with anti-corrosion material.</p>	18 months
SR 3.8.4.5	<p>Verify battery connection resistance meets the limits specified in Table 3.8.4-1.</p>	18 months
SR 3.8.4.6	<p>Verify:</p> <p>a. Each required 125 V battery charger supplies ≥ 200 amps at ≥ 125 V for ≥ 4 hours; and</p> <p>b. Each required 250 V battery charger supplies ≥ 200 amps at ≥ 250 V for ≥ 4 hours.</p>	18 months

(continued)

Table 3.8.6-1 (page 1 of 1)
Battery Cell Parameter Requirements

PARAMETER	CATEGORY A: LIMITS FOR EACH DESIGNATED PILOT CELL	CATEGORY B: LIMITS FOR EACH CONNECTED CELL	CATEGORY C: LIMITS FOR EACH CONNECTED CELL
Electrolyte Level	> Minimum level indication mark, and ≤ ¼ inch above maximum level indication mark ^(a)	> Minimum level indication mark, and ≤ ¼ inch above maximum level indication mark ^(a)	Above top of plates, and not overflowing
Float Voltage	≥ 2.13 V	≥ 2.13 V	> 2.10 V
Specific Gravity ^{(b)(c)}	≥ 1.205	≥ 1.200 <u>AND</u> Average of all connected cells > 1.205	Not more than 0.020 below average of all connected cells <u>AND</u> Average of all connected cells ≥ 1.205

- (a) It is acceptable for the electrolyte level to temporarily increase above the specified maximum level during and following equalizing charges provided it is not overflowing.
- (b) Corrected for electrolyte temperature and level. Level correction is not required, however, when on float charge and battery charging current is < 2 amps.
- (c) A battery charging current of < 2 amps when on float charge is acceptable for meeting specific gravity limits following a battery recharge, for a maximum of 7 days. When charging current is used to satisfy specific gravity requirements, specific gravity of each connected cell shall be measured prior to expiration of the 7 day allowance.



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 239 TO

RENEWED FACILITY OPERATING LICENSE NO. DPR-46

NEBRASKA PUBLIC POWER DISTRICT

COOPER NUCLEAR STATION

DOCKET NO. 50-298

1.0 INTRODUCTION

By application dated October 29, 2010 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML103080079), as supplemented by letters dated June 10 and August 31, 2011 (ADAMS Accession Nos. ML11167A083 and ML11250A160, respectively), Nebraska Public Power District (the licensee), submitted a license amendment request (LAR) in which it requested changes to the Technical Specifications (TSs) for Cooper Nuclear Station (CNS). The amendment would revise the acceptance criteria in CNS TS 3.8.4, "DC [Direct Current] Sources - Operating," Surveillance Requirement (SR) 3.8.4.1, and TS 3.8.6, "Battery Cell Parameters," Table 3.8.6-1, "Battery Cell Parameter Requirements." Specifically, amendment would revise the acceptance criteria in TS SR 3.8.4.1 and TS Table 3.8.6-1 by revising the battery terminal voltage on float charge and specific gravity (SG) acceptance criteria to ensure that the safety-related batteries can perform their safety functions and remain operable during postulated design basis events.

The supplemental letters provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on January 25, 2011 (76 FR 4386).

2.0 REGULATORY EVALUATION

2.1 System Description

The electrical power system at CNS consists of various alternating current (AC) and DC systems. Two of the DC power systems are the 125 Volt (V) system and the 250 V system. These two systems provide both motive and control power to the selected safety-related and non-safety-related equipment. The systems are designed to have sufficient independence, redundancy, and testability to perform their safety functions, assuming a single failure.

Each of these systems provides two independent on-site sources of DC power for startup operation, shutdown, and the loads required for station safety. The loss of any one source will not prevent safe shutdown of the station. The safety objective of these two systems is to provide an uninterruptible source of power to supply normal and emergency 125 V DC and 250 V DC control and power loads under all conditions.

The 125 V DC and 250 V DC systems each have two subsystems. The Division I and II 125 V DC subsystems each consist of a 125 V battery, battery charger, and distribution system. The Division I and II 250 V DC subsystems each consist of a 250 V battery, battery charger, and distribution system. Each 125 V battery has 58 individual cells and each 250 V battery has 120 individual cells.

2.2 General Requirements

Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The TSs ensure the operational capability of structures, systems, and components that are required to protect the health and safety of the public. The NRC's regulatory requirements related to the content of the TSs are contained in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.36, "Technical specifications," which requires that the TSs include items in the following specific categories: (1) safety limits, limiting safety systems settings, and limiting control settings; (2) limiting conditions for operations; (3) SRs; (4) design features; and (5) administrative controls. The regulations in 10 CFR 50.36(c)(3) specify that SRs are "requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met."

The regulations in 10 CFR 50.63, "Loss of all alternating current power," require that each light-water-cooled nuclear power plant licensed to operate must be able to withstand for a specified duration and recover from a station blackout (SBO).

General Design Criterion (GDC) 17, "Electric power systems," of Appendix A¹, "General Design Criteria for Nuclear Power Plants," to Part 50 of 10 CFR requires, in part, that,

An onsite electric power system and an offsite electric power system shall be provided to permit functioning of structures, systems, and components that are important to safety. The safety function for each system (assuming the other system is not functioning) shall be to provide sufficient capacity and capability to assure that (1) specified acceptable fuel design limits and design conditions of the reactor coolant pressure boundary are not exceeded as a result of anticipated operational occurrences and (2) the core is cooled and containment integrity and other vital functions are maintained in the event of postulated accidents.

The onsite electric power supplies, including the batteries, and the onsite electric distribution system, shall have sufficient independence, redundancy, and testability to perform their safety functions assuming a single failure.

¹ The 1967 Proposed GDC as described in the CNS updated safety analysis report, Appendix F, are the licensing basis for CNS; however, the NRC staff concluded in its 1973 Safety Evaluation Report for CNS that the intent of the 1971 Final Rule for 10 CFR Part 50, Appendix A, had also been met.

The regulations in Appendix R, "Fire Protection Program for Nuclear Power Facilities Operating Prior to January 1, 1979," to 10 CFR Part 50 require, in part, that:

During the postfire shutdown, the reactor coolant system process variables shall be maintained within those predicted for a loss of normal [AC] power, and the fission product boundary integrity shall not be affected; i.e., there shall be no fuel clad damage, rupture of any primary coolant boundary, or rupture of the containment boundary.

NRC Regulatory Guide (RG) 1.155, "Station Blackout," August 1988 (ADAMS Accession No. ML003740034), describes an acceptable alternative method for complying with NRC regulations in 10 CFR 50.63.

Nuclear Management and Resources Council, Inc. (NUMARC) 87-00, "Guidelines and Technical Bases for NUMARC Initiatives Addressing Station Blackout at Light Water Reactors," November 1987, provides guidance and methodologies for implementing SBO initiatives.

The Institute of Electrical and Electronics Engineers Standard 485-1983, "IEEE Standard for the Sizing of Large Stationary Type Power Plant and Substation Lead Storage Batteries," describes methods for defining the DC load and for sizing a lead-acid battery to supply that load for stationary battery applications in full float operations.

While CNS is not currently committed to NRC RG 1.129, Revision 2, "Maintenance, Testing, and Replacement of Vented Lead-Acid Storage Batteries for Nuclear Power Plants," February 2007 (ADAMS Accession No. ML063490110), the NRC staff used this document as a technical reference during its review of this amendment. RG 1.129, Revision 2, describes a method that the NRC staff considers acceptable for use in complying with the agency's regulations with regard to the maintenance, testing, and replacement of vented lead-acid storage batteries in nuclear power plants.

3.0 TECHNICAL EVALUATION

3.1 Proposed Changes

Current TS SR 3.8.4.1 states:

SURVEILLANCE		FREQUENCY
SR 3.8.4.1	Verify battery terminal voltage on float charge is: a. \geq 125 V for the 125 V batteries; and b. \geq 250 V for the 250 V batteries.	7 days

Revised TS SR 3.8.4.1 would state:

SURVEILLANCE	FREQUENCY
SR 3.8.4.1 Verify battery terminal voltage on float charge is: a. ≥ 125.9 V for the 125 V batteries; and b. ≥ 260.4 V for the 250 V batteries.	7 days

Current TS Table 3.8.6-1, regarding Specific Gravity, states:

PARAMETER	CATEGORY A: LIMITS FOR EACH DESIGNATED PILOT CELL	CATEGORY B: LIMITS FOR EACH CONNECTED CELL	CATEGORY C: LIMITS FOR EACH CONNECTED CELL
Specific Gravity ^{(b)(c)}	≥ 1.195	≥ 1.190 <u>AND</u> Average of all connected cells > 1.200	Not more than 0.020 below average of all connected cells <u>AND</u> Average of all connected cells ≥ 1.190

Revised TS Table 3.8.6-1, regarding Specific Gravity, would state:

PARAMETER	CATEGORY A: LIMITS FOR EACH DESIGNATED PILOT CELL	CATEGORY B: LIMITS FOR EACH CONNECTED CELL	CATEGORY C: LIMITS FOR EACH CONNECTED CELL
Specific Gravity ^{(b)(c)}	≥ 1.205	≥ 1.200 <u>AND</u> Average of all connected cells > 1.205	Not more than 0.020 below average of all connected cells <u>AND</u> Average of all connected cells ≥ 1.205

The licensee also proposed to make conforming changes to the TS Bases.

3.2 NRC Staff Evaluation

CNS Updated Safety Analysis Report (USAR), Chapter VIII, "Electrical Power Systems", Section 6.0, "125/250 Volt DC Power Systems," describes the design of the safety-related 125/250 V DC power systems. CNS USAR Chapter VIII, Section 6.2, "Safety Design Bases," states that each 125 V and 250 V safety-related battery must have adequate capacity to safeguard the station until AC power sources are restored. The 125/250 V safety-related batteries must provide power for maintaining the plant in a safe hot shutdown condition in the event control room operation is prevented by fire and the alternate shutdown system is used. The 125/250 V Division I and II safety-related batteries must provide power for a 4-hour duration during an SBO in accordance with 10 CFR 50.63, NUMARC 87-00, and RG 1.155. In addition, the Division II 125 V and 250 V safety-related batteries are required to provide power for a 4½-hour duration in accordance with Appendix R to 10 CFR Part 50.

SR 3.8.4.1 requires verification of battery terminal voltage while on float charge to ensure the adequacy of the charging system and the ability of the batteries to perform their intended safety function. Based on its evaluation and calculation, the licensee determined that the current battery terminal voltage on float charge acceptance criteria in the TS SR 3.8.4.1 is non-conservative. At CNS, the battery cells are designed for a nominal electrolyte SG of 1.215. The corresponding minimum cell float voltage for the nominal SG of 1.215 is 2.17 V per cell. The licensee's evaluation determined that the correct battery terminal voltage on float charge would be 125.9 V and 260.4 V, respectively, based on the total number of battery cells (i.e., 58 cells in a 125 V DC subsystem and 120 cells in a 250 V DC subsystem) and the battery design data. The NRC staff reviewed the above evaluation and concludes that the proposed acceptance criteria for battery terminal voltage on float charge will ensure that the battery is maintained in a fully-charged state. Maintaining the battery in a fully-charged state provides assurance that the battery will be capable of performing its safety functions. Based on the above, the NRC staff concludes that the proposed change to TS SR 3.8.4.1 is acceptable.

The licensee stated that the current CNS TS Table 3.8.6-1 includes non-conservative battery electrolyte SG acceptance criteria. Specifically, the current TS Table 3.8.6-1 SG values (≥ 1.195 for Category A, ≥ 1.190 and > 1.200 for Category B, and ≥ 1.190 for Category C) do not represent the limits necessary to demonstrate that sufficient capacity exists for the safety-related batteries to perform their intended safety functions. The proposed values (≥ 1.205 for Category A, ≥ 1.200 and > 1.205 for Category B, and ≥ 1.205 for Category C) have been incorporated into the licensee's DC Load Flow Analysis. The licensee stated that the consideration of the correct SG limits resulted in an average SG value (1.205) less than the low end (1.210) of the nominal SG range (1.215 ± 0.005) that required a derating of the battery capacity from 100 percent to 96 percent. According to the licensee, this reduced battery capacity of 96 percent is already included in the design margin factor consideration in the battery-sizing calculation. By letter dated June 10, 2011, in response to the NRC staff's request for additional information (RAI) dated May 13, 2011 (ADAMS Accession No. ML111320511), the licensee stated that it consulted the battery manufacturer who provided an engineering estimate of battery capacity of 96 percent of rated capacity based on the average SG. The licensee analyzed both divisions of the 125 V and 250 V DC safety-related systems using nuclear-qualified Electrical Power Systems Design, Simulation and Analytic (EDSA) software and concluded that the proposed SG values would ensure that the 125 V and 250 V DC safety-

related systems would be able to meet electrical requirements and perform their safety functions to ensure safe shutdown and for maintaining the plant in safe shutdown mode. In addition, the licensee also evaluated SBO scenarios and Appendix R fire scenarios (Division II only for Appendix R) and found the scenarios bounded by the load flow analyses and the proposed SG values. By letter dated August 31, 2011, the licensee provided a copy of the battery manufacturer's estimate of the battery capacity based on the average SG. The NRC staff verified that the battery capacity would be 96 percent using the average SG values.

The CNS safety-related batteries are sized in accordance with Institute of Electrical and Electronics Engineers Standard 485-1983, "IEEE Standard for the Sizing of Large Stationary Type Power Plant and Substation Lead Storage Batteries." Based on the CNS battery design, the minimum required battery capacity is 90 percent or greater of the manufacturer nameplate rating. Since the proposed TS SG acceptance limits correspond to battery capacity that is higher than the minimum CNS required capacity, the NRC staff concludes that the proposed change to TS Table 3.8.6-1 will restore conservatism to the battery SG acceptance limits and will ensure that the safety-related batteries will perform their safety functions. Based on the above, the NRC concludes that the proposed change to TS Table 3.8.6-1 is acceptable.

3.3 NRC Staff Conclusion

The NRC staff reviewed the licensee's proposed changes to CNS TS SR 3.8.4.1 for battery terminal voltage on float charge and TS Table 3.8.6-1 for the SG acceptance criteria. The NRC staff concludes that the proposed changes to CNS TS SR 3.8.4.1 and TS Table 3.8.6-1 provide reasonable assurance of the continued availability of the required DC electrical power to shut down the reactor and to maintain the reactor in a safe condition after an anticipated operational occurrence or postulated design-basis events. The amendment corrects the non-conservative minimum battery float voltage acceptance criteria in TS SR 3.8.4.1 and the non-conservative battery electrolyte SG acceptance criteria in TS Table 3.8.6-1. The staff concludes that the proposed changes are conservative, consistent with the CNS design basis, and consistent with the battery manufacturer's recommendation and are, therefore, acceptable. Furthermore, the NRC staff concludes that the proposed TS changes are in accordance with 10 CFR 50.36 and 10 CFR 50.63, and meet the intent of GDC 17, and the guidance in RG 1.129, RG 1.155, and NUMARC 87-00. Therefore, the NRC staff concludes that the proposed changes are acceptable.

The NRC staff has no objection to the licensee's proposed changes to the TS Bases.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Nebraska State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no

significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* on January 25, 2011 (76 FR 4386). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: P. Sahay

Date: October 28, 2011

October 28, 2011

Mr. Brian J. O'Grady
Vice President-Nuclear and CNO
Nebraska Public Power District
72676 648A Avenue
Brownville, NE 68321

SUBJECT: COOPER NUCLEAR STATION - ISSUANCE OF AMENDMENT RE: NON-CONSERVATIVE BATTERY TERMINAL VOLTAGE ON FLOAT CHARGE AND SPECIFIC GRAVITY ACCEPTANCE CRITERIA IN TECHNICAL SPECIFICATION (TAC NO. ME4974)

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A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

/RA/

Lynnea E. Wilkins, Project Manager
Plant Licensing Branch IV
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-298

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1. Amendment No. 239 to DPR-46
2. Safety Evaluation

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ADAMS Accession No. ML112850265

*memo dated

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NAME	LWilkins	JBurkhardt	RElliott	RMathew (A)	RHarper	MMarkley	LWilkins
DATE	10/13/11	10/13/11	10/19/11	9/29/11	10/21/11	10/28/11	10/28/11

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