

10 CFR 50.73

October 7, 2011
BW110112

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Braidwood Station, Units 1 and 2
Facility Operating License Nos. NPF-72 and NPF-77
NRC Docket Nos. STN 50-456 and STN 50-457

Subject: Withdrawal of Licensee Event Report 2011-003-00, "Drained Sections of Piping in Auxiliary Feedwater Suction Lines Result in System Inoperability Due to Inadequate Technical Evaluation"

Reference: Letter from Daniel J. Enright (Exelon Generation Company, LLC) to U.S. NRC, "Licensee Event Report 2011-003-00 - Drained Sections of Piping in Auxiliary Feedwater Suction Lines Result in System Inoperability Due to Inadequate Technical Evaluation" dated May 31, 2011

The referenced Licensee Event Report (LER) involved a condition in the Auxiliary Feedwater (AF) pump suction lines. These lines were originally configured during construction to contain a drained section (i.e., dry pipe) between two suction valves as a means to prevent possible leakage of raw essential service water (SX) into the AF system and subsequently, inadvertent injection into steam generators. The effect of this air void or dry pipe on the AF pumps' operability was not adequately justified with supporting design documentation; therefore, the drained sections of the suction lines were filled to provide assured operability pending further evaluations.

To address the historic operability of the AF system, Exelon Generation Company, LLC (EGC) performed comprehensive full-scale testing using a pump and suction line configuration identical in critical parameters to the installed configuration at Braidwood. The tests covered different flow rates, void volumes, suction pressures, and pump status. The results of this controlled testing demonstrated the AF pumps would have performed their design function without a decrease in performance or damage to the pump from the air void ingestion.

October 7, 2011
U.S. Nuclear Regulatory Commission
Page 2

Since reasonable assurance of operability was demonstrated via this testing, EGC has determined the condition no longer satisfies any 10 CFR 50.73 reporting requirements. Accordingly, the referenced LER is being withdrawn.

There are no regulatory commitments contained in this letter. Should you have any questions concerning this submittal, please contact Mr. Chris VanDenburgh, Regulatory Assurance Manager, at (815) 417-2800.

Respectfully,

A handwritten signature in dark ink, appearing to read "Daniel J. Enright", with a stylized flourish at the end.

Daniel J. Enright
Site Vice President
Braidwood Station

DJE/vak

cc: NRR Project Manager – Braidwood Station
Illinois Emergency Management Agency – Division of Nuclear Safety
US NRC Regional Administrator, Region III
US NRC Senior Resident Inspector (Braidwood Station)
Illinois Emergency Management Agency - Braidwood Rep