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NRC FORM 195 U.S. NUCLEAR REGULATORY COMMON						DOCKET NUMBER 50-263	
NRC DISTRIBUTIO		FILE NUMBER INCIDENT REPORT					
TO:	FROM: Northern States Power Company Minneapolis, Minn.			D/	ATE OF DOCUMENT		
Mr. J. G. Keppler				D	8/11/76 DATE RECEIVED 8/13/76		
		L. O. Mayer					
		рнор		INPUT FORM		NUMBER OF COPIES RECEIVED One signed copy	
DESCRIPTION	<u></u>	A	ENCLOS	URE			
Ltr. trans the followin	g :		8/5/	76 concerning re educed core flow	educti V.	50-263-76-12) on on in MAPLHGR limits	
		(1-P)	(1-P	(1-P) ACKING			
PLANT NAME: Monticello		NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS $\mathcal{W}_{\mathcal{V}}$					
······		FOR ACTION	 /INFORM	ATION 8/16/7	6	RJL	
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LPDR:Minneapolis, Minn.		DISTRIBUTION				CONTROL NUMBER	
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IORTHERN STATES POWER COMPANY

MINNEAPOLIS, MINNESOTA 55401

Regulatory Reduct Fils

August 11, 1976

Mr J G Keppler, Director, Region III Office of Inspection & Enforcement U S Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

Dear Mr Keppler:

MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

Reduction in MAPLHGR Limits at Reduced Core Flow

The Licensee Event Report for this occurrence is reproduced on the back of this letter. Enclosed are 3 copies.

Yours very truly,

L O Mayer, PE Manager, Nuclear Support Services

LOM/MHV/deb

cc: Director, IE, USNRC (40) Director, MIPC, USNRC (3) G Charnoff MPCA Attn: J W Ferman



8263

LICENSEE EVENT REPORT									
	CONTROL BLOCK			(PLEASE PRINT ALL RE	QUIRED INFORMATION)				
				LICENSE TYPE 4 1 1 1 1 1 26 30					
	ATEGORY REPORT REPORT TYPE SOURCE TYPE SOURCE T 58 59 60 61	DOCKET NUMBER	6 3 0 8 68 69	EVENT DATE 3 0 5 7 6 0 74 7	REPORT DATE 0 8 1 1 7 6 5 80				
	DESCRIPTION cic ECCS Model Analysis	for BUR-3's	indicates 220	OF PCT may be	wooded at				
7 8 9	ced core flows. A 5% r				80				
				is is completed.	80				
	ious occurrences.	ICIMUE MILLI I	(M-R0-76		80				
7 8 9			S		80				
7 8 9 System CODE 07 S F 7 8 9 10	CAUSE CODE B ZZZZZZ 11 12				80				
	DESCRIPTION educed flows where the	onset of trạn	sition boilin	g could occur, c	onservative				
7 8 9 09 heat	transfer coeff's. are	applied which	decrease hea	t transfer away	BO from clad and				
7 8 9 10 incr	ase PCT.				80				
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FORM ACTIVI RELEAS 7 8 9 PEPSON	OF CONTENT	F ACTIVITY	NA45	LOCATION OF REL	_				
	ER TYPE DESCRIPTION				80				
NUMB	NEL INJURIES R DESCRIPTION 0 NA 11 12				80				
OFFSITE	CONSEQUENCES								
16 Z	R DAMAGE TO FACILITY DESCRIPTION NA		·						
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17 NA 7 8 9					80				
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	*		,		80				
7 8 9		D F Novinalii		<u>~12/205 5</u>	80				
1 - C.	NAME:	D E Nevinski		PHONE:	GP0 881+667				



ORTHERN STATES POWER COMP. Monticello Nuclear Generating Plant Monticello, MN 55362

August 6, 1976

Mr. J. G. Keppler, Director Region 111 Office of Inspection and Enforcement United States Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

Dear Mr. Keppler;

MONFICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

On August 5, 1976, we were notified by General Electric Co. of an analytical phenomenon generic to BWR-3's in the ECCS model for boiling water reactors. At reduced core flow there is a point for which the model calculates resultant peak clad temperature in excess of the 2200°F limit during a LOCA.

To prevent this potentiality, a 5% reduction in allowable Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) at core flow below 85% of rated has been administratively implemented. This will continue until further EXCS Nodel analysis is completed.

Yours very truly,

tern K C.C.R.a.

8/6/26

L. R. Eliason Plant Manager

LRE/sdd

Rec'd 1:31

cc: Director, Office of Management Information & Program Control G. Charnoff Minnesota Pollution Control Agency Attn: J W Ferman File

Copy sent by telecopier to Keppler