

ROUTING AND TRANSMITTAL SLIP		ACTION	
TO (Name, office symbol or location) IE Chief, FOSB IE Chief, FC&EB IE:HQ(4)	INITIALS	CIRCULATE	
	DATE	COORDINATION	
Licensing(4) DR Central Files J. Rizzo, OMIPC C. Poslusny, PSB	INITIALS	FILE	
	DATE	INFORMATION	
Region I Region II PDR Local PDR	INITIALS	NOTE AND RETURN	
	DATE	PER CONVERSATION	
MSIC TIC OGC, Beth, P-506A A. Roisman	INITIALS	SEE ME	
	DATE	SIGNATURE	
REMARKS			
<p>NORTHERN STATES POWER COMPANY MONTICELLO PLANT DOCKET NO. 50-263</p> <p>Attached is a copy of licensee's reply dated June 2, 1976, to IE Bulletin 76-04.</p>			
Do NOT use this form as a RECORD of approvals, concurrences, disapprovals, clearances, and similar actions			
FROM (Name, office symbol or location) G. Fiorelli, IE:III	DATE	06-15-76	
	PHONE		

OPTIONAL FORM 41  
AUGUST 1967  
GSA FPMR (41CFR) 100-11.206

648-10-81594-1 552-103 GPO 5041-101

# NSP

NORTHERN STATES POWER COMPANY 1976

MINNEAPOLIS, MINNESOTA 55401

In view of increased "in service" inspection requirements of the reactor, as well as more appropriate surveillance June 2, 1976. If the material is not available, it would be necessary to have the material and proper documentation made available.

In the time period prior to completion of this request for information, the following methods and existing methods are being used to provide information about the reactor system. (See attached) which provide  
**Mr. James G. Keppler**  
Director - Region III  
Office of Inspection and Enforcement  
United States Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

## MONTICELLO NUCLEAR GENERATING PLANT

Docket No. 50-263 License No. DFR-22

The following material is submitted in response to IE Bulletin No. 76-04, "Cracks in Cold Worked Piping at BWR's."

A survey has been performed to develop the specific listing of stainless steel piping as defined in L. 2 of the bulletin. The piping surveyed is that greater than two inches nominal size, exposed to reactor coolant pressure during operation, and inside the second isolation valve of the reactor coolant system.

Because documentation requirements at the time the Monticello Plant was constructed were not as completely formalized as they are today, the task of verifying the solution annealing of cold worked material has required considerable effort. The specific listing has been prepared and developed to include, in each case, the purchase order, manufacturer, material and purchase specification, and heat number. In most cases, however, it has been necessary to request specific documentation from the supplier or manufacturer to verify solution annealing. The specifications do require the process in all instances, but documentation is not specific enough to conclude that the annealing was done. Completion of the solution annealing verification is continuing. It is anticipated that the work will be completed by August, 1976.

The program now in progress will identify any cold worked austenitic stainless steel which may not have been solution annealed. Adverse findings will be reported to the NEC Regional Office within 24 hours as requested. Individual consideration will be given to any material identified as not having had proper annealing to determine if the appropriate action should

JUN 10 1976

