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MCGAUGHEY,R.W.            Iowa Electric Light & Power Co.  
RECIP.NAME                 RECIPIENT AFFILIATION  
DENTON,H.R.                Office of Nuclear Reactor Regulation, Director

**SUBJECT:** Requests permanent relief from individual valve leakage testing requirements of ASME Boiler & Pressure Vessel Code Section XI for torus-drywell vacuum breakers, Fee enc1.

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Iowa Electric Light and Power Company

August 9, 1984  
NG-84-3082

Mr. Harold Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Subject: Duane Arnold Energy Center  
Docket No: 50-331  
Op. License No: DPR-49  
Inservice Testing Program - Torus/Drywell  
Vacuum Breaker Leak Testing

- References:
1. IE Letters dated March 1, 1978, revised May 14, 1980 (Revision 2), revised November 11, 1980 (Revision 3), and revised December 8, 1982 (Revision 4).
  2. Enclosure 1 of NRC Letter dated September 26, 1983 from D. B. Vassallo (NRC) to L. Liu (IE).
  3. Letter, R. McGaughy (IE) to H. Denton (NRC) dated March 29, 1984 (NG-84-1269)

Dear Mr. Denton:

By Reference 1, Iowa Electric Light and Power Company (IE) submitted a proposed inservice testing (IST) program description and request for relief from testing requirements, for the Duane Arnold Energy Center (DAEC). One request (VR-11) sought to obtain relief from individual valve leakage testing requirements of the ASME Boiler and Pressure Vessel Code Section XI for the seven torus-drywell vacuum breakers.

Reference 2 provided results of the NRC staff review of our relief requests. For VR-11, NRC granted relief but indicated that "Although the licensee provided justification for not meeting the Code, permanent relief is not granted at this time...pending supplemental information." NRC requested that we investigate methods (plant modifications, etc.) of determining individual torus-drywell vacuum breaker leakage.

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Mr. Harold Denton  
August 9, 1984  
NG-84-3082  
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The attachment to this letter summarizes the results of our investigation to determine individual vacuum breaker leak testing methods (Reference 3). We have determined that, although not necessary for safety, methods of determining individual torus-drywell vacuum breaker leakage are technically feasible. However, each method results in increased radiation exposure to personnel, increased personnel safety concerns, and increased risk of damage to the vacuum breakers and torus internals. As discussed in the attachment, we believe the present DAEC method of determining torus-drywell vent system leakage provides both a conservative and positive means of verifying that the total bypass leakage does not exceed the maximum allowable limits, thus assuring acceptable individual vacuum breaker performance. In light of the foregoing, we are requesting that permanent relief, in accordance with VR-11 of Reference 1, be granted.

In accordance with the requirements of 10 CFR Part 170, we are enclosing the required application fee of \$150.

If you have any questions, please feel free to call me.

Very truly yours,

*R. W. McGaughy*  
Richard W. McGaughy  
Manager, Nuclear Division

RWM/MSG/dbm\*

Attachment: 1. Torus-Drywell Vacuum Breaker Leak Testing Options

cc: M. Grim  
L. Liu  
S. Tuthill  
M. Thadani (NRC)  
NRC Resident Office  
Commitment Control No. 84-0082

DUANE ARNOLD ENERGY CENTER  
TORUS-DRYWELL VACUUM BREAKER  
LEAK TESTING OPTIONS

## 1.0 INTRODUCTION

By reference 1, Iowa Electric Light and Power Company (IE) submitted a proposed inservice testing (IST) program description and request for relief from testing requirements for the Duane Arnold Energy Center (DAEC). One request (VR-11) sought to obtain relief from the individual valve leakage testing requirements of the ASME Boiler and Pressure Vessel Code Section XI for the seven torus-drywell vacuum breakers.

Reference 2 provided results of the NRC staff review of Iowa Electric's relief requests. For VR-11, NRC granted relief but indicated that "Although the licensee provided justification for not meeting the Code, permanent relief is not granted at this time...pending supplemental information". NRC requested that IE investigate methods of determining individual torus-drywell vacuum breaker leakage.

We have completed our investigation, and have herein summarized the evaluation results for determining individual torus-drywell vacuum breaker leakage.

## 2.0 DESIGN AND SAFETY FUNCTION OF TORUS-DRYWELL VACUUM BREAKERS

The DAEC torus-drywell vacuum breaker group consists of seven vacuum breakers located inside the torus. Each vacuum breaker (Figure 1) is bolted to a short run of 18" piping which is welded to the torus-drywell vent header. Each vacuum breaker is provided with the counterbalanced pallet and a magnet providing positive closure after valve operation.

The primary safety function of the drywell-torus breaker group is to prevent drywell pressure from being significantly less than torus pressure. A second group, the reactor building-torus group prevents torus pressure from being significantly less than reactor building pressure.

In the event that drywell pressure becomes significantly less than torus pressure, the torus-drywell vacuum breakers open allowing noncondensable gases to flow from the torus air space to the drywell. Vacuum breaker leakage does not affect the ability to perform this safety function.

A second safety-related function associated with the torus-drywell vent system and vacuum breakers is containment pressure suppression. In the event of a process system piping failure within the drywell, reactor water and steam will be released into the drywell gas space.

The resulting increased drywell pressure forces a mixture of noncondensable gases, steam and water into the torus-drywell vent system. The torus-drywell vacuum breakers remain closed, forcing the mixture into the pressure suppression pool. The steam condenses rapidly in the suppression pool resulting in a pressure reduction in the containment.

Torus-drywell vent system leakage can allow the mixture of non-condensable gases, steam, and water to flow directly into the torus gas space bypassing the suppression pool (called bypass leakage). Because steam which bypasses the suppression pool may not be condensed, excessive bypass leakage may affect containment pressure suppression.

Chapter 6 of the DAEC Updated Final Safety Analysis Report discusses the maximum allowable bypass leakage for the DAEC containment in detail. It is shown there that the maximum allowable leakage area is approximately  $0.2 \text{ ft}^2$ , which corresponds to the area of a 6 inch pipe. This maximum allowable bypass leakage represents the sum of all drywell to torus gas space leakage paths, including torus-drywell vacuum breakers. Based on this, individual vacuum breaker leakage would not adversely affect pressure suppression as long as the total bypass leakage does not exceed the maximum allowable limit.

### 3.0 PRESENT TESTING METHOD

Iowa Electric presently conducts a leak test of the drywell vent system (vent pipes, headers, downcomers, and vacuum breakers) at the end of each regularly scheduled refueling outage, before the pressurization of the primary system. The present testing method is summarized in the following:

- Step 1: Pressurize drywell so that its pressure is 1-1.25 psi higher than the torus pressure.
- Step 2: If Step 1 pressurization cannot be achieved, excessive leakage exists. The leakage source is identified and repaired; and a new leak test is begun.
- Step 3: After a positive pressure differential of  $\Delta 1-1.25 \text{ psi}$  between the drywell and torus is achieved, the pressure of the torus is monitored. The torus pressure response must be such that its corresponding "calculated by-pass area" is less than that of a 1" orifice.
- Step 4: If the corresponding "calculated by-pass area" exceeds that of a 1" orifice, the source of leakage is identified and repaired.

The 1" orifice leakage area, used as the acceptance criteria for the present leak test of the drywell-torus vent system, represents only 3% of the maximum allowable bypass leakage area for the DAEC containment.

Additionally, this method provides conservative results because testing is performed at low drywell pressure, and the torus-drywell vacuum breakers increase in leak tightness with increased pressure on the drywell side. Consequently, the present method of testing the vent system and the test acceptance criteria ensure that individual vacuum breaker leak rates are acceptable. The DAEC method of testing the torus-drywell vacuum breakers is also consistent with the methods used at other BWRs.

#### 4.0 INDIVIDUAL VACUUM BREAKER TESTING METHODS

As directed by the NRC in Reference 2, methods for performing individual testing of the seven torus-drywell breakers have been investigated. The following methods have been identified and evaluated:

- Bench Testing of Vacuum Breakers
- Use of Inflatable Plugs for In-Place Testing of Vacuum Breakers
- Use of Standard Pipe Plugs for In-Place testing of Vacuum Breakers
- Installation and use of Block Valves for In-Place Testing of Vacuum Breakers
- Use of Plexiglas/Aluminum Covers for In-Place Testing of Vacuum Breakers

Although any of the above methods is technically feasible, we found they all possess each of the following disadvantages:

- Increased radiation exposure of approximately 20 Man-rem per refueling outage to plant personnel due to working within the torus
- Increased personnel safety hazards due to working within the torus
- Increased risk of damage to the vacuum breakers and torus internals due to individual leak test equipment and procedures.

#### 5.0 CONCLUSIONS

Leakage between the drywell and torus gas space (bypass leakage) must be minimized because it can affect containment pressure suppression capability. Torus-drywell vacuum breaker leakage contributes to total bypass leakage, and therefore, must be controlled. However, determination of individual torus-drywell vacuum breaker leakage is not necessary to verify containment pressure suppression capability. Verification that total bypass leakage does not exceed the maximum allowable is adequate.

Although not necessary for safety, methods of determining individual torus-drywell vacuum breaker leakage are technically feasible. However, each method results in increased radiation exposure to personnel, increased personnel safety concerns, and increased risk of damage to vacuum breakers and torus internals.

Based on the foregoing, we believe the NRC staff will agree that leak testing individual torus-drywell vacuum breakers is technically feasible but may not be desirable when plant and personnel safety is taken into consideration. We also believe the present DAEC method of determining torus-drywell vent system leakage provides a conservative and positive means of verifying that total bypass leakage does not exceed the maximum allowable limit and assures individual vacuum breaker leakage performance is acceptable.

#### 6.0 REFERENCES

1. IE Letters dated March 1, 1978, revised May 14, 1980 (Revision 2), revised November 11, 1980 (Revision 3), and revised December 8, 1982 (Revision 4).
2. Enclosure 1 of NRC Letter dated September 26, 1983 from D. B. Vassallo (NRC) to L. Liu (IELP).

Item No.	Part Number	Qty	Description	ASME OR AIBI Material	Material and Non-Destructive Exam.					Notes	
					C	H	N	UT	M	P	
1.1	LC-240-797	1	Pallet Seat Assembly	(LC-240-778)			X	X	X		8
1.1.1	LC-240-778	1	Seat (Mach)	(LC-240-778)							7,8,9
1.1.1.1	LD-240-734	1	Seat Flange (Mach)	(LB-240-748)							3,8
1.1.1.2	LB-240-748	1	Flange	M350-LF1	X	X	X				9,6
1.1.1.3	LC-240-762	1	Seat Ring (Mach/Weld)	(LC-240-749)							4,9
1.1.1.4	LC-240-749	1	Seat Ring (Form)	(LC-240-774-3)							8
1.1.1.5	LA-240-750	1	Seat Ring	SAS16 GR. 70	X	X	X				8
1.1.1.6	LC-240-774-3	1	Hinge Ear	(LC-240-774-5)							8
1.1.1.7	LC-240-774-5	2	Hinge Ear	SAS16 GR. 70	X	X	X				8
1.1.1.8	LA-240-709	1	A/R Weld Rod	SFA5.1/E7018	X	X	X				9
1.1.1.9	LA-240-709	1	A/R Weld Rod	SFA5.9/ER309	X						9
1.2	LC-240-751	1	Pallet Assm.	(LC-240-587)							7,8,9
1.2.1	LC-240-708	1	Pallet (Mach)	(LC-240-774-2)			X				3,8
1.2.2	LC-240-587	1	Pallet (Form)	(LC-240-774-2)							5,9
1.2.2.1	LB-240-720	1	Hinge Arm #1 (Mach)	(LB-240-749)	X	X	X				8
1.2.2.2	LB-240-719	1	Hinge Arm #1 (Mach)	(LB-240-749)	X	X	X				9,6
1.2.2.3	LB-240-720	1	Hinge Arm #2 (Mach)	(LB-240-721)	X	X	X				8,8
1.2.2.4	LB-240-721	1	Hinge Arm #2 (Mach)	(LB-240-721)	X	X	X				8,8
1.2.2.5	LA-240-616-2	4	Stems 3/8"	(LA-240-903)							8
1.2.2.6	LA-240-602	4	1/8" Dia. Rounds	303 S.S.							2,9
1.2.2.7	SP-1003-2	4	Hex Nut 3/8"	18-8 S.S.							2,9
1.2.2.8	LA-240-783-1	4	Stud 1/4"	(LA-240-804)							8
1.2.2.9	LA-240-783-2	16	Stud 1/4"	(LA-240-804)							8
1.2.2.10	LA-240-804	4/R	1/4" Dia. Rounds	303 S.S.							2,9
1.2.2.11	BP-388-A	1	A/R Silicone Sealant	Dow #781							1
1.2.2.12	LC-240-784	1	Seal	(SP-1020-8)							1,2,9
1.2.2.13	LC-240-753	1	Seal Retainer (Mach)	(LC-240-804)							8
1.2.2.14	LB-240-775	1	Seal Retainer (Rolled)	304 S.S.							2,9
1.2.2.15	LB-240-806	1	Magnet Plate	410 S.S.							8
1.2.2.16	BP-1003-V	17	Hex Nut 1/4-28	18-8 S.S.							2,9
1.2.2.17	SP-1000-AF	1	Hex Cap Screw 1/4-28	303 S.S.							2,9
1.2.2.18	SP-332-E	16	Washer	18-8 S.S.							2,9
1.2.2.19	SP-322-F	2	Lock Washer	18-8 S.S.							2,9
1.2.2.20	LA-240-809	2	Hinge Bushing	(LA-240-765)							1,8
1.2.2.21	LA-240-785	1	R 1" Dia. Rounds	Bronze							2,9
1.2.2.22	LA-240-10	1	Hinge Shaft	(LA-240-801)							303 S.S.
1.2.2.23	LA-240-801	1	A/R 9/16 Dia.	18-8 S.S.							2,9
1.2.2.24	SP-331-HG	4	Set Screw	Ph15-7 Mo S.S.							1,2,9
1.2.2.25	LA-240-32	2	Retainer Ring	303 S.S.							8
1.2.2.26	LA-240-620	2	Adjuster Stud	303 S.S.							8
1.2.2.27	LA-240-741	2	Adjuster Stud	303 S.S.							8
1.2.2.28	LA-240-780-1	1	O-Ring	Ethylene Propylene							1,2,9
1.2.2.29	LA-240-901	1	A/R Retaining Compound	Locite RC 40							—
2.1	LC-240-851	1	Housing (Weld)	—							3,7,8
2.2	LB-240-763	1	Flange (Mach)	(LB-240-764)	X	X	X				9
2.3	LB-240-766	1	Body (Weld/Mach)	(LB-240-765)	X	X	X				8,9
2.4	LC-240-769-1	1	Body Reducer 18 X 30	A516 GR. 70							9
2.5	LC-240-770	1	Reducer (Elliptical)	(LC-240-774-4)							5,9
2.6	LC-240-774-6	1	Reducer	AS350 GR. 70							2,9,6
2.7	LC-240-782	1	18" Weld Neck Flange	AS350 GR. LF1							9
2.8	LA-240-731	1	3/4" N.P.T. Drain Conn	(LA-240-805)							9
2.9	LA-240-543	1	Lock Ear (Form)	(LC-240-774-4)							8
2.10	LA-240-774-4	3	Lock Ear	AS316 GR. 70							8
2.11	LA-240-829	1	1" Sch. 80 Soc. Weld	(LA-240-805)							8
2.12	LA-240-805	1/R	2-1/4 Dia X 12-1/2 Lg.	AS350 GR. LF1	X	X	X				9,6
2.13	LA-240-774-7	2	Mounting Foot	(LC-240-774-7)							—
2.14	LA-240-733-1	2	Mounting Foot	SAS16 GR. 70	X	X	X				—
2.15	LC-240-532	1	Hinge Post (Mach)	(LC-240-931)							—
2.16	LC-240-531	1	Hinge Post (Weld)	(LC-240-931)							—
2.17	LC-240-230	1	Hinge Post (Screw)	SA285 GR.C							—
2.18	LC-240-915	1	Actuator Arm Assem.	—							—
2.19	LC-240-217	1	Actuator Arm (Mach)	(LC-240-916)							—
2.20	LC-240-916	1	Actuator Arm (Blank)	SA285 GR.C							—
2.21	LA-240-169	1	Pivot Hub	304 S.S.							—
2.22	LA-240-190	1	Roller Hub	304 S.S.							—
2.23	LA-240-19	1	Shaft 1/2"	303 S.S.							—
2.24	LA-240-650	1	Bearing	Bronze							—
2.25	LA-240-445	2	Boiler	303 S.S.							—
2.26	LA-240-541-1	2	Retainer Ring 1/2"	Ph15-7 Mo S.S.							—
2.27	LA-240-651	2	Bearing	Bronze							—
2.28	LA-240-20	1	Shaft	303 S.S.							—
2.29	LA-240-561-2	2	Retainer Ring 5/8"	Ph15-7 Mo B.S.							—
2.30	LA-240-561	2	Lock Washer	18-8 S.S.							—
2.31	LA-240-787	1	Magnet	Alnico Gr. V							—
2.32	LA-240-518	1	Magnet Spacer	2011-T3 ALUM							—
2.33	LA-240-732	1	Magnet Spacer	(SP-1020-B)							—
2.34	LA-240-565	3	Shim 1/32	304 S.S.							—
2.35	LA-240-623	3	Shim 1/64	304 S.S.							—
2.36	LA-240-537-1	1	Socket Hd. Screw 7/16	18-8 S.S.							—
2.37	SP-330-A	4	Hd. Hd. Screw	18-8 S.S.							—
2.38	SP-332-O	4	Plain Washer	H							