

Dominion Nuclear Connecticut, Inc.
Millstone Power Station
Rope Ferry Road
Waterford, CT 06385



DominionSM

AUG 18 2011

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

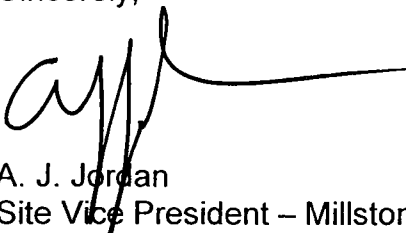
Serial No. 11-411
MPS Lic/LES R0
Docket No. 50-336
License No. DPR-65

DOMINION NUCLEAR CONNECTICUT, INC.
MILLSTONE POWER STATION UNIT 2
LICENSEE EVENT REPORT 2011-002-00
MILLSTONE POWER STATION UNIT 2 REACTOR TRIP ON LOW STEAM GENERATOR
LEVEL

This letter forwards Licensee Event Report (LER) 2011-002-00 documenting an event that occurred at Millstone Power Station Unit 2 on June 20, 2011. This LER is being submitted pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in a manual or automatic actuation of any of the systems listed in 50.73(a)(2)(iv)(B), including the Reactor Protection System and the Auxiliary Feedwater Actuation System.

If you have any questions or require additional information, please contact Mr. William D. Bartron at (860) 444-4301.

Sincerely,



A. J. Jordan
Site Vice President – Millstone

Attachments: 1

Commitments made in this letter: None

IEZZ
NRK

cc: U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
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C. J. Sanders
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NRC Senior Resident Inspector
Millstone Power Station

ATTACHMENT

LICENSEE EVENT REPORT 2011-002-00

**MILLSTONE POWER STATION UNIT 2
DOMINION NUCLEAR CONNECTICUT, INC.**

1. FACILITY NAME Millstone Power Station - Unit 2	2. DOCKET NUMBER 05000336	3. PAGE 1 OF 2
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4. TITLE
Reactor Trip on Low Steam Generator Level

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
06	20	2011	2011 - 002 - 00			08	18	2011	FACILITY NAME	DOCKET NUMBER
										05000
										05000

9. OPERATING MODE 1	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)										
	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)							
10. POWER LEVEL 059	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)							
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)							
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)							
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)							
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER							
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)								

Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME William D. Barton, Nuclear Station Licensing	TELEPHONE NUMBER (Include Area Code) 860-444-4301
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED	15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

At 1152 on June 20, 2011, with Millstone Power Station Unit 2 operating at 59 percent power in Mode 1, the reactor tripped automatically on low steam generator water level. The decreasing water level condition was due to a low suction pressure trip on the "B" steam generator feed pump (SGFP) that occurred while placing the "A" SGFP in service. The event is being reported pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in a manual or automatic actuation of any of the systems listed in 50.73(a)(2)(iv)(B). The actuation of the Auxiliary Feedwater Actuation System also is a reportable condition under the same paragraph.

The cause of the event was gaps in the application of operator fundamentals and some procedure quality issues associated with operations procedure OP 2204, Load Changes.

Procedure OP 2204 has been revised to ensure the second steam generator feedwater pump is placed in service at a lower power level. Additional corrective actions to address the underlying causes of the gaps in the application of operator fundamentals are being addressed in accordance with the station's corrective action program.

LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
Millstone Power Station - Unit 2	05000336	YEAR	SEQUENTIAL NUMBER	REV NO.	2 OF 2
		2011	- 002 -	00	

NARRATIVE

1. Event Description

At 1152 on June 20, 2011, with Millstone Power Station Unit 2 operating at 59 percent power in Mode 1, the reactor tripped automatically on low steam generator (SG) water level 300 microseconds prior to a manual reactor trip initiated from the control room. The decreasing SG water level condition was due to a low suction pressure trip on the operating "B" steam generator main feedwater pump (SGFP) [SJ]. The "B" SGFP low suction pressure trip occurred while attempting to place the "A" steam generator main feedwater pump in service. Established operating procedures had recently been changed to permit starting the second SGFP at a higher power level than previously performed.

Auxiliary Feedwater (AFW) [BA] initiated as expected following loss of the operating SGFP. Standard post trip actions were carried out, and all other safety systems responded as expected.

The event is being reported pursuant to 10 CFR 50.73(a)(2)(iv)(A) as an event that resulted in a manual or automatic actuation of any of the systems listed in 50.73(a)(2)(iv)(B), including the Reactor Protection System. The actuation of the Auxiliary Feedwater Actuation System also is a reportable condition under the same paragraph.

2. Cause

The cause of the event was gaps in the application of operator fundamentals and some procedure quality issues associated with operations procedure OP 2204, Load Changes.

3. Assessment of Safety Consequences

The safety consequences of this event were low. The reactor automatically tripped on low steam generator level. Steam generator levels remained in the visible range on narrow range SG level instrumentation. All other safety systems responded as expected.

There was no loss of decay heat removal capability, because main and auxiliary feedwater pumps were available to feed the steam generators. Neither departure from nucleate boiling nor fuel centerline melt design limits were challenged. As such, there were no challenges to the fuel, reactor coolant system or containment fission product barriers.

4. Corrective Action

Procedure OP 2204 has been revised to ensure the second steam generator feedwater pump is placed in service at a lower power level. Additional corrective actions to address the underlying causes of the gaps in the application of operator fundamentals are being addressed in accordance with the station's corrective action program.

5. Previous Occurrences

No previous similar events/conditions were identified.

Energy Industry Identification System (EIIS) codes are identified in the text as [XX].