ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9105130049 DOC.DATE: 91/05/03 NOTARIZED: NO DOCKET # FACIL:50-331 Duane Arnold Energy Center, Iowa Electric Light & Pow 05000331 AUTH.NAME AUTHOR AFFILIATION MINECK,D.L. Iowa Electric Light & Power Co. RECIP.NAME RECIPIENT AFFILIATION MURLEY,T.E. Office of Nuclear Reactor Regulation, Director (Post 870411 R

D

S

1

Α

R

I

D

S

A

D

D

S

SUBJECT: Submits semiannual rept for plan for integrated scheduling of plant mods for facility.NRC item on Schedule B, "Vendor Info Project (Generic Ltr 90-03);Phase 1, 'Vendor Info Control Procedure'" completed on 910412.

DISTRIBUTION CODE: IE47D COPIES RECEIVED:LTR / ENCL / SIZE: TITLE: 50.59 Annual Report of Changes, Tests or Experiments Made W/out Approv

NOTES:

	RECIPIENT ID CODE/NAME	COPII LTTR	ES ENCL	RECIPIENT ID CODE/NAME	COP LTTR	IES ENCL	D
	PD3-3 LA	1	0	PD3-3 PD	5	5	D
		T	U				
INTERNAL:	ACRS	6	6	AEOD/DOA	1	1	S
	NRR/DLPQ/LHFB11	1.	1	NRR/DOEA/OEAB11	1	1	-
e	REG FILE 02	1	. 1 .	RGN3 FILE 01	1	1	
EXTERNAL:	NRC PDR	1	1	NSIC	1	·1	

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 20 ENCL 18

Iowa Electric Light and Power Company

May 3, 1991 NG-91-0966

Dr. Thomas E. Murley Director of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Mail Station P1-137 Attention: Document Control Desk Washington, D.C. 20555

Subject: Duane Arnold Energy Center Docket No.: 50-331 Op. License No.: DPR-49 Semiannual Report for the "Plan for the Integrated Scheduling of Plant Modifications for the Duane Arnold Energy Center"

D. Mineck letter to T. Murley dated November 5, 1990

Reference:

NG-90-2457 File: A-278

Dear Dr. Murley:

This letter and attachments provide the semiannual report required by Section V.A. of the "Plan for the Integrated Scheduling of Plant Modifications for the Duane Arnold Energy Center" (the Plan). This report summarizes our progress in implementing Schedule A and B items, identifies the changes since the last report, summarizes the reasons for schedule changes, and provides updated schedules.

Attachment 1 is a summary of progress in implementing the items listed in Schedules A and B. It lists the seven items which have been completed since the last update (Referenced).

Attachment 2 identifies the changes since the last report. These revisions include changes to the Item Descriptions of nine items, the revision of nineteen Schedule commitment dates, the addition of one Schedule A item and two Schedule B items. Two Schedule B items have been deleted since our last report. The reasons for the schedule changes are stated in Attachment 3.

Updated Schedules A and B are included as Attachment 4. For each item listed, the specific implementation date is stated and reference is made to the NRC correspondence supporting this date. When available, references to NRC correspondence are cited for IELP-initiated items listed in Schedule B. Also, brief descriptions of those Schedule B items, which are not specifically described in other correspondence are included as Attachment 5.

9105130049 910503 FDR ADOCK 05000331

PDR

General Office • P.O. Box 351 • Cedar Rapids, Iowa 52406 • 319/398-4411

Dr. Thomas Murley NG-91-0966 May 3, 1991 Page 2

Please note that one NRC item on Schedule B "Vendor Information Project (GL 90-03); Phase 1: Vendor Information Control Procedure" was completed on April 12, 1991. Also, one Iowa Electric (IELP) Initiative Item on Schedule B "Service Water System Enhancements; River Sediment Management System" has been delayed. The reason for the schedule change is explained in Attachment 3. These items were not completed by their scheduled dates. Your staff was notified promptly upon discovery of the overdue items.

Please inform us if you have any questions or comments concerning this submittal.

Very truly yours,

Daniel L. Mineck Manager, Nuclear Division

DLM/DJM/pjv

Attachments:

- nts: 1. Summary of Progress in Implementing Schedule A and B Items 2. Changes in Schedules
 - 3. Summary of Reasons for Schedule Changes
 - 4. Updated Schedules A and B

5. Descriptions of Selected Schedule B Items

D. MICHKE					
L. Liu					
L. Root					
R. McGaughy					
S. P. Sands(NRC-NRR)					
A. Bert Davis (Region I)	II)				
NRC Resident Office					
Commitment Control Nos.	870105,	880169,	890405,	890406.	890408
	890409	890410	900122.	900123	900126
	900127	900346	900347	900354.	900355
· .	900361	,	,	· · · ,	

Attachment 1 NG-91-0966 May 3, 1991 Page 1 of 1

Summary of Progress in Implementing Schedule A and B Items

The following items, as listed on the Schedules A and B transmitted with the November 5, 1990 semiannual report, have been completed during this reporting period:

<u>Schedule A</u>

There were no items listed on Schedule A in the November 5, 1990 Semi-Annual Update.

<u>Schedule</u> B

- Emergency Diesel Generator (EDG) Reliability Program
- Vendor Information Project (GL 90-03)

Phase 1: Vendor Information Control Procedure

- Implement Bill of Materials (BOM) Program
- Drill/Exercise Utilization of Control Room Simulator

Permanent Use of Simulator

- Reliability Centered Maintenance (RCM) Pilot Study
- Technical Specifications Improvement

Short-term Enhancements

Phase 1

Spent Fuel Improvements/Study

Attachment 2 NG-91-0966 May 3, 1991 Page 1 of 7

Changes in Schedules

Changes to the Schedules fall into four categories: revised descriptions of items, revised completion dates, addition of new items and deletion of items.

I. Item Descriptions

Current Item Description

Revised Item Description

Schedule A

There were no items listed on Schedule A in the November 5, 1990 Semi-Annual Update.

Schedule B

• Inservice Testing Program Modifications (GL 89-04)

- Inservice Testing Program Modifications (GL 89-04)
 - Phase 1: Residual Heat Removal (RHR) and River Water Supply (RWS) Valve Modifications
 - Phase 2: Diesel Generator Air-Operated Solenoid Valve Replacement
 - Phase 3: Diesel Fuel Oil Transfer System Modifications
- Safety-Related MOV Operability/Testing (GL 89-10)
 - Phase 1: Completion of Design Review and Development of Test Plan
 - Phase 2: Completion of Baseline Static Testing
- Phase 3: Implementation and Continued Use o Testing Program

• Safety-Related MOV Operability/Testing GL (89-10)

Attachment 2 NG-91-0966 May 3, 1991 Page 2 of 7

Current Item Description

• Control Building HVAC and Chillers

Phase 2: Control Building (Battery Room) HVAC Enhancements

• Verification of Safety-Related HVAC Ductwork

• Configuration Management

• Scram Frequency Reduction

•Long-term Instrument and Control Strategy

Digital Feedwater Control Analog Trip System Program Study

Revised Item Description

•Control Building HVAC and Chillers (Note: Phase 2 has been cancelled)

Phase 3: Chiller Refurbishment

•Operability Determination of Seismically Supported HVAC Ductwork

• Configuration Management

Program Plan

Digital Imagining (DI)

Phase 1: Establishment of Operational Drawings on DI platform

Phase 2: Establishment of Balance of Plant Drawings of DI platform

• Scram Frequency Reduction

Mechanical Feedwater Modifications

Digital Feedwater Control

- Electro-Hydraulic Control (EHC) System Improvements
- •Long-term Instrument and Control Strategy

Analog Trip System Program Study

Attachment 2 NG-91-0966 May 3, 1991 Page 3 of 7

Current Item Description

•Metrology Lab

• Self-Initiated Safety System Functional Inspection (SSFI)

Revised Item Description

•Metrology Lab

Phase 1: Construction of Lab Facility

Phase 2: Equipment Installation and Lab Operational

• Self-Initiated Instrument and Control System Functional Inspection (I&CSFI)

Attachment 2 NG-91-0966 May 3, 1991 Page 4 of 7

Current Revised Description Completion Date Completion Date Schedule A There were no items listed in Schedule A in the November 5, 1990 Semi-Annua Schedule B • Security System Upgrades Security Computer System Upgrade December 31, 1992 June 30, 1993 June 30, 1993	•
Schedule A There were no items listed in Schedule A in the November 5, 1990 Semi-Annua Schedule B • Security System Upgrades Security Computer System Upgrade December 31, 1992 June 30, 1993 Access Control Upgrade	_
There were no items listed in Schedule A in the November 5, 1990 Semi-Annua <u>Schedule B</u> •Security System Upgrades Security Computer System Upgrade December 31, 1992 Access Control Upgrade December 31, 1992 June 30, 1993 June 30, 1993	
<u>Schedule B</u> • Security System Upgrades Security Computer System Upgrade December 31, 1992 Access Control Upgrade December 31, 1992 June 30, 1993	l Update.
• Security System Upgrades Security Computer System Upgrade December 31, 1992 June 30, 1993 Access Control Upgrade December 31, 1992 June 30, 1993	
December 31, 1992 Oune 30, 1993	
•Station Blackout Rule Compliance	
Procedure changes June 30, 1991 Note 3	-
•Emergency Response Capabilities (Supplement 1 to NUREG-0737)	
Regulatory Guide 1.97 Schedule not yet September 1, 19 certain, evaluation NG-91-0640 of scope in progress (NG-89-0057)	991/
•Inservice Testing Program Modifications (GL 89-04)	
Phase 1: Residual Heat Removal July 5, 1991 July 5, 1991 (RHR) And River Water Supply (RWS) Valve Modifications	
Phase 2: Diesel Generator Air- July 5, 1991 December 31, 19 Operated Solenoid Valve Replacement	991
Phase 3: Diesel Fuel Oil July 5, 1991 Prior to Cycle Transfer Systems Startup Modifications	12

Attachment 2 NG-91-0966 May 3, 1991 Page 5 of 7

Description		Current <u>Completion Date</u>	Revised <u>Completion Date</u>				
•Safety-Re Testing (lated MOV Operability/ GL 89-10)						
Phase 1:	Completion of Design Review and Development of Test Plan	Prior to Cycle 13 Startup	December 31, 1991				
Phase 2:	Completion of Baseline Static Testing	Prior to Cycle 13	December 31, 1992				
Phase 3:	Implementation and Continued Use of Testing Program	Prior to Cycle 13 Startup	June 28, 1994				
•Verificat Adequacy Electrica	ion of Seismic of Mechanical and l Equipment	Prior to Cycle 13 Startup/NG-88-3209 to NRC	Prior to Cycle 14 Startup				
•Core Stab (BN-88-07)	ility Studies	June 30, 1991	December 31, 1991				
•Emergency System (Gl	Response Data 89-15)	June 30, 1991	December 31, 1991				
•Service W Enhancemer River Sec Managemer	ater System its liment it System	March 31, 1991	September 30, 1991				
•Configurat	tion Management	· .					
Program F	lan	Note 2	December 31, 1991				
Digital I	maging (DI)						
Phase 1:	Establishment Operational Drawings on DI platform	Note 2	March 31, 1992				
Phase 2:	Establishment of Balance of Plant Drawings on DI Platform	Note 2	September 30, 1992				

Attachment 2 NG-91-0966 May 3, 1991 Page 6 of 7

Description	Current <u>_Completion Date</u>	Revised <u>Completion Date</u>
Scram Frequency Reduction		
Digital Feedwater Control	Prior to Cycle 12	Prior to Cycle 13

• Facility Upgrade Study

• Scram

Note 2

December 31, 1991

Attachment 2 NG-91-0966 May 3, 1991 Page 7 of 7

III. New Items

Description

Schedule A

•Revision to DAEC Radiation Protection Program to reflect new 10 CFR Part 20 Requirements.

Schedule B

IELP Initiative Items

•Computer Software Quality Assurance

•Maintenance Self-Assessment

January 1, 1993

Completion Date

Schedule not yet certain.

Schedule not yet certain

IV. Items Deleted

Description

Reason for Deletion

Schedule A

There were no items listed on Schedule A in the November 5, 1990 Semi-Annual Update.

Schedule B

- Steam Leak Detection
 - Phase 2: Evaluation of Isolation Setpoints
- Control Building HVAC and Chillers
 - Phase 2: Control Building (Battery Room) HVAC Enhancements

This project has been cancelled. A cost benefit analysis has shown that this project is not cost effective, <u>i.e.</u>, it is more economical to replace the batteries periodically than to upgrade the battery room cooling system.

This project has been included

as part of the Design Basis

Document program.

Attachment 3 NG-91-0966 May 3, 1991 Page 1 of 3

Summary of Reasons for Schedule Changes

Item Description	Change	Explanation
<u>Schedule A</u>		
There were no items on Schedule	A in the November	5, 1990 Semi-Annual Update.
<u>Schedule B</u>		
•Security System Upgrades Security Computer System Upgrade	December 31, 1992 to June 30, 1993	The schedule has been revised to avoid performing
Access Control Upgrade	December 31, 1992 to June 30, 1993	during the cycle 12 refueling outage.
•Station Blackout Rule Compliance		
Procedure changes	June 30, 1991 to Note 3	Schedule for Procedural Changes required by
		be determined 30 days after issuance of NRC Safety Evaluation Report (SER).
•Emergency Response Capabilities (Supplement 1 to NUREG-0737)		
Regulatory Guide 1.97	Schedule not yet certain, evaluation of scope in progress (NG-89-0057).	These schedules have been revised to reflect project milestones that were provided in a letter from D. Mineck(IELP)to Dr. Murley (NRC) on April 12, 1991 (NG-91-0640).
•Inservice Testing Program Modifications (GL 89-04)	July 5, 1991 to the following milestones	The schedule has been revised to reflect project milestones that were provided in a letter
	Phase 1: July 5, 1991	from D. Mineck (IELP) to Dr. Murley (NRC) on January 5, 1991 (NG-90-0030).
	Phase 2: December 31, 1991	
· · · · · · · · · · · · · · · · · · ·	Phase 3: Prior to Cycle 12 Startup	

Attachment 3 NG-91-0966 May 3, 1991 Page 2 of 3

		raye 2 01 3
Item Description	<u>Change</u>	Explanation
•Safety-Related MOV/ Operability Testing (GL 89-10)	Prior to Cycle 13 startup to the following milestones	The schedule has been revised to to incorporate specific project milestones that have been identified since the last
	Phase 1: December 31, 1991	update of the integrated plan.
	Phase 2: December 31, 1992	
	Phase 3: June 28, 1994	
 Verification of Seismic Adequacy of Mechanical and Electrical Equipment (USI A-46, GL 87-02) 	Prior to Cycle 13 Startup to Prior to Cycle 14 Startup	We have changed the completion date of this project due to delays in the issuance of the Generic Implementation Procedure (GIP), Revision 2, Safety Evaluation Report (SER).
•Core Stability Studies	June 30, 1991 to December 31, 1991	The schedule has been revised to reflect delays in issuance of the BWR Owners' Group specific Licensing Topical Report.
•Emergency Response Data System (GL 89-15)	June 30, 1991 to December 31, 1991	The schedule has been revised to reflect unforeseen technical difficulties in establishing the computer link between

• Service Water System Enhancements

River Sediment Management System March 31, 1991 to September 30 1991 Weather-related construction delays and unforeseen design issues have been encountered.

our computers and NRC

computers.

Attachment 3 NG-91-0966 May 3, 1991 Page 3 of 3

Item Description		Change	Explanation
•Configurat	tion Management		
Program Plan Digital Imaging (DI)		Note 2 to	The schedule has been
		December 31, 1991	specific project milestones
Phase 1:	Establishment of Balance of Plant Drawings on DI Platform	Note 2 to March 31, 1992	since the last update of the integrated plan.
Phase 2:	Establishment of Balance of Plant Drawings on DI Platform	Note 2 to September 30, 1992	
•Scram Free	quency Reduction		The schedule has been revised
Digital Fe	edwater Control	Prior to Cycle 12 startup to Prior to Cycle 13 startup	feedwater system to determine if installation of digital feedwater control would be cost effective.

•Facility Upgrade Study

Note 2 to December 31, 1991

The project schedule has been established.

Attachment 4 - Page 1 of 6 NG-91-0966 May 3, 1991

110y 0, 1991	1991 1992 1993 1 99 4 1995 1 99
	$[v_{c}]_{e} = 11 - c_{v_{c}} = 12 - c_{v_{c}} = 13 - c_{v_{c}} = 14$
SCHEDULE A	
Revision to DAEC Radiation Protection Program to reflect new 10 CFR 20 Requirements	•January 1, 1993
SCHEDULE B	
NRC Items	
 Security System Upgrades 	
Security Computer System Upgrade	•June 30, 1993
Access Control Upgrade	•June 30, 1993
• Station Blackout Rule Compliance	
Schedule Submittal required by 10 CFR 50.63(c)(4)	(Note 3)
RCIC Turbine Insulation	Prior to Cycle 12 Startup/NG-90-0757 to NRC
Control Room Lighting Improvements	•Prior to Cycle 12 Startup/NG-90-0757 to NRC
Procedure Changes	(Note 3)
• Containment Performance Improvements	
Hardened Wetwell Vent (GL 89-16)	•December 31, 1992/
Other Containment Performance Improvements	NG-89-2886 to NKC (Note 4)

Attachment 4 - Page 2 of 6 NG-91-0966 May 3, 1991

	1991 1992 1993 1994 1995	1996
SCHEDULE B (Continued)	Cycle 11- Des Cycle 12- Cycle 13- Cycle 13-	Cycle 14
 Emergency Response Capabilities (Supplement 1 to NUREG-0737) 		
Detailed (Supplement 1) CRDR		
Phase 4 Long Term Enhancements	Prior to Cycle 12 Startup/ NG-86-4251 to NRC	
Regulatory Guide 1.97	•September 1, 1991/NG-91-0640 to NRC	
 Individual Plant Examination 		
Initial Level I PRA	•June 30, 1991/NG-89-3096 to NRC	
Containment Performance Analysis	•June 30, 1992/ NG-89-3096 to NRC	
Report Submittal	•August 31, 1992/ NG-89-3096 to NRC	•
 Inservice Testing Program Modifications (GL 89-04) 		
Phase 1: Residual Heat Removal (RHR) and River Water Supply (RWS)	•July 5, 1991/NG-90-0030 to NRC	
Phase 2: Diesel Generator Air-Operated	•December 31, 1991/NG-90-0030 to NRC	
Phase 3: Diesel Fuel Oil Transfer System Modifications	Prior to Cycle 12 Startup/NG-90-0030 t	o NRC

Attachment 4 - Page 3 of 6 NG-91-0966 May 3, 1991 1992 1993 1994 1995 1996 1991 Cycle 11-----Cýcle 12----Cycle 13------Cycle 13------Cvcle 14 SCHEDULE B (Continued) • Safety-Related MOV Operability/Testing (GL 89-10) Phase 1: Completion of Design ----•December 31, 1991 Review and Development of Test Plan Phase 2: Completion of Baseline ----- December 31, 1992 Static Testing -----20, 1994 Phase 3: Implementation and Continued use of Testing Program • Verification of Seismic Adequacy of ---•Prior to Cycle 14 Mechanical and Electrical Equipment Startup (USI A-46, GL 87-02) -----June 30, 1992 • Off-site Dose Assessment Manual (ODAM) Revision Schedule ----•December 31, 1991 • Core Stability Studies (BN 88-07) • Vendor Information Project (GL 90-03) Phase 2: Verification of Safety---•June 15, 1991/NG-90-2289 Related Equipment Vendor Manuals Verification of Non-Safety-----•December 21, 1991 Phase 3: Related Equipment Vendor Manuals

Attachment 4 - Page 4 of 6 NG-91-0966 May 3, 1991

	1991 I	1992		1993	. 1	1994	ļ	19	995		199	6
						·			ĺ			
SCHEDULE B (Continued)	Cycle 11		Cycle	12		Cyc 1	e 13		4	- Сус	:le 14	
IELP Initiative Items			•			•						
 Additional 161kV Service Breaker 		Prio	or to C	ycle	12 Sta	artup						
 Control Building HVAC and Chillers 												
Phase 3: Chiller Refurishment		-•Decémbe	r 31,	1991								
 Operability Determination of Seismically Supported HVAC Ductwork 	•June	30, 1991/	'NG-90-1	0318		ι.						
 Design Basis Program Phase I - ECCS and Selected Safety- Related Systems 		an an an an an ar an an ar an ar	. Ber ann ann ann ann ann ann a	au au 72 - au	•Sej	ptembe	er 30,	, 1993	, , }		۲.,	
 Emergency Response Data System (GL 89-15) 		-•Decembe	r 31,	1991					••••			
 Shielding Study for Revised Hydrogen Water Chemistry Program 		•Prio	or to C	ycle	12 Sta	artup						
 Hydrogen Water Chemistry Oxygen/Hydrogen Generator 	(Note 2)	* .								•	
• Plant Life Extension	(Note 2) .				· ·						
• Power Systems Analysis		-•Decembe	r 31,	1991							· •	
• Severe Accident Management	(Note 1)						•				
• Technical Specifications Improvement						• .						
Short-term Enhancements			·						, ·			
Phase 2:		-•Decembe	r 31, 1	1991								. '
Long-term Improvements	(Note 1)	,									

Attachment 4 - Page 5 of 6 NG-91-0966 May 3 1991

May 3, 1991	1,991 199 2 1993	1994	1995	1996
SCHEDULE B (Continued)	Cycle 11	Cycle 13	Сус	le 14
 Ultrasonic Examination of Reactor Vessel Beltline Region Welds 		· . · · · ·		
Phase 2: Feasibility Study, Position Paper and Drawing Development	•June 30, 1991	•		
Phase 3: Vessel Weld Examination	(Note 2)			
• Telemetry for Emergency Sirens		December 31,	1993	
 Comprehensive Procurement Initiative 			,	
Evaluation Implmentation	•July 1, 1991 •July 1, 1992			· •
 Service Water System Enhancements 				
River Water Supply Pumps River Sediment Management System	•September 30, 1991	December 31,	1993	
 Replace Two Electro-Hydraulic Control (EHC) Pumps 	Prior to Cycle 12 St	artup		- -
• Configuration Management Plan				
Program Plan	•December 31, 1991			• • •
Digital Imaging			,	
Phase 1: Establish of Operational Drawings on DI platform	•March 31, 1992		т.,	
Phase 2: Establishment of Balance of Plant Drawings on DI Platform	•September 30,	1992		· . · ·

Attachment 4 - Page 6 of 6 NG-91-0966 May 3, 1991

	1991 199	2 1993	1994	1995	1996
SCHEDULE B (Continued)	Cycle 11	——Cycle 12 — 🍽	Cycle 13-	Cyc	le 14
 Scram Frequency Reduction 				, ,	1. 1
Mechanical Feedwater Modifications	•P	rior to Cycle 12	Startup		
Digital Feedwater Control			Prior to Cycl	e 13 Startup	
Turbine Electro-Hydraulic Control (EHC) System Improvements				•Prior to	o Cycle 14 Startup
 Long-term Instrument & Control Strategy 					· · ·
Analog Trip System Program Study	(Note 2)	:	•		
• Metrology Lab				•	· · · ·
Phase 1: Construction of Lab Facility Phase 2: Equipment Installation and Lab Operational	(Note 2) (Note 2)				
• Facility Upgrade Study	•Dece	mber 31, 1991	· · ·		
 Self-initiated Instrument & Control System Functional Inspection (I&CSFI) 	(Note 2)				
• Computer Software Quality Assurance	(Note 2)	•			
• Maintenance Self-Assessment	(Note 2)			•	
Note 1: Schedule not yet certain, await Note 2: Schedule not yet certain, potent Note 3: Schedule not yet certain. Sched with 10 CFR 50.63(c)(3). Note 4: Schedule not yet certain. Poten Plant Evaluation as requested by	ing promulgation tial IELP initia dule due within tial improvemen Generic Letter	of NRC requireme tive item. 30 days of the no ts to be evaluate 89-16.	nts/guidance. tification pr d during Indi	ovided in ac vidual	cordance

Attachment 5 NG-91-0966 May 3, 1991 Page 1 of 11

Description of Selected Schedule B Items.

I. <u>NRC Items</u>

Security System Upgrades

Security Computer System and Access Control Update

The Physical Security Computer System (PSCS) project consists of upgrading the main security computer hardware, the Central Alarm Station/Secondary Alarm Station (CAS/SAS) operator's consoles, access control, and the Video Switching Sub-system (VSS). The major functions provided by the PSCS will be access control, alarm monitoring and annunciation, security record storage and report generation, security material issue control, and simulator/training.

The PSCS will be a real time, database management system utilizing distributed intelligent processing at both multiplexers and remote control panels.

A new video switching and control system will be provided and will interface with existing fixed and pan/tilt/zoom (PTZ) cameras. The video switching system will become a subsystem to the PSCS and will receive inputs from the host security computers. These inputs will be processed into the proper form for alarm camera/monitor call-up, Close-Circuit Television Camera (CCTV) sequencing, and provide control outputs as required to operate camera equipment and auxiliary functions.

Station Blackout Rule Compliance

RCIC Turbine Insulation

The RCIC Turbine is being insulated in order to maintain the RCIC room temperature at or below acceptable levels during a Station Blackout event.

Control Room Lighting Improvements

This project involves upgrading DC-powered overhead lighting in order to improve Control Room illumination during a Station Blackout.

Procedure Changes

The project involves reviewing and revising our Station Blackout Procedures to provide operators with better guidance during a Station Blackout event. Additionally, System Control Center Procedures are being reviewed and revised to ensure that they are in agreement with changes in the operations procedures to ensure that site power is restored as quickly as possible.

Attachment 5 NG-91-0966 May 3, 1991 Page 2 of 11

Emergency Response Capabilities

<u>Phase 4</u>

Completion of Phase 3 resolved all safety significant Human Engineering Deficiencies (HEDs). Phase 4 will incorporate human factor considerations in areas such as:

- 1. Rod Block Monitors and Recorders
- 2. Condenser vacuum breakers
- 3. Lighting and Switch locations
- 4. Indicator scales
- 5. Control room atmospheric monitoring
- 6. Control room annunciator panels
- Inservice Testing Program Modifications

<u>Phase 1:</u> <u>Residual Heat Removal (RHR) and River Water Supply (RWS) Valve</u> <u>Replacement</u>

The piping and associated isolation valves between the Residual Heat Removal (RHR) system and the High Pressure coolant Injection (HPCI) system which make-up the steam-condensing mode of RHR are currently "abandoned-in-place." Although these isolation valves have been electrically-disabled in the isolated position, they must still be periodically stroke-time and leakage rate tested. This proposed modification will remove these valves and connecting piping and blank flange the remaining connections, thereby, eliminating the need to stroke-time and leak rate test the isolation valves.

The valve control logic for two solenoid valves (SV-4934 and SV-4935) in the River Water Supply system will be modified so that the the valves can be individually stroke tested, as required, without temporarily disabling the valve logic.

Phase 2: Diesel Generator Air-Operated Solenoid Valve Replacement

This project involves the periodic replacement or refurbishment of these solenoid valves under the DAEC's Maintenance Program for solenoid valves.

<u>Phase 3: Diesel Fuel Oil Transfer System Modifications</u>

The piping for the diesel fuel oil transfer system will be modified to permit full-flow testing of the diesel fuel oil transfer pump discharge check valves. This modification will provide instrument taps downstream of the check valves to permit flow measurement through the check valves.



Off-Site Dose Assessment Manual (ODAM) Revision Schedule

The purpose of this project is to address discrepancies and implement improvements to the ODAM that were identified in the NRC Safety Evaluation of the ODAM (reference: Letter J. Hall (NRC) to L. Liu (IELP) dated August 17, 1990).

<u>Core Stability Studies</u> (BN#88-07)

We are pursuing a plant-specific study on core thermal-hydraulic stability in conjunction with two other BWRs of comparable core size. The purpose of this study is to demonstrate that, due to the relatively small size of the DAEC core, unique options are available for addressing the NRC Bulletin. Additionally, we are following developments in the BWR Owners' Group (BWROG) study of this subject.

Vendor Information Project (GL 90-03)

Phase 2: Verification of Safety-Related Equipment Vendor Manuals

A review will be conducted to assure that up-to-date vendor manuals are available for safety-related equipment. The manual for each item of safety-related equipment will be identified and recorded in a computer data base. The vendor will be contacted, where possible, to either verify that our library has the current revision of that manual or to obtain the correct revision.

<u>Phase 3:</u> Verification of Non-Safety-Related Equipment Vendor Manuals

The same effort described in Phase 2 for safety-related equipment will be completed for non-safety-related equipment.

II. <u>IELP INITIATIVE ITEMS</u>

Additional 161KV Service Breaker

The purpose of this project is to:

- Eliminate a single-failure which results in a Loss Of Off-Site Power (LOOP).
- 2. Reduce maintenance and repair restrictions on the East 161KV bus and startup transformer feeder breaker.
- Increase reliability of the startup transformer feed to a breaker-and-a-half* scheme which is consistent with all the other 161 KV loads and lines.

*A breaker-and-a-half scheme consists of three breakers in series between two main buses with two circuits connected between the three breakers. This arrangement allows for a circuit to be taken out of service for maintenance purposes while still supplying power to the other circuit. In normal operation both circuits are energized.

Attachment 5 NG-91-0966 May 3, 1991 Page 4 of 11

Control Building HVAC and Chillers

Phase 3: Chiller Refurbishment

This project involves the rebuilding of the Control Building Chillers (A & B). The refurbishment is being done to improve the chillers' reliability and to facilitate ease of maintenance functions. This includes the retubing of the evaporator and condenser, installation of a new compressor and upgrading of associated electrical equipment and piping, as required.

Operability Determination of Seismically Supported HVAC Ductwork

This project is a visual examination of seismically mounted HVAC ductwork. Based on data gathered during a visual inspection of the ductwork, an operability assessment of the ductwork supports will be completed to document the ductworks ability to function.

<u>Design Basis Program</u>

Phase I-ECCS and Selected Safety Systems

The intent of establishing a Design Basis Program is to organize and collate the design bases information with supporting design information that provides for each system, structure, or component the rationale or "whys" for their functional requirements and controlling parameters. The design bases include assumptions, numerical values and other information used in the design of any system, structure, or component to assure that functional and regulatory goals are met. Design bases will be stated in concise terms and will focus on the specific functions or bounding parameters of each system, structure, or component. To this end, the objectives of the Design Basis Program are to assemble information concerning structures, systems, and components which are important to safety through:

- 1. an organized review of functional requirements and controlling (bounding) parameters for each structure, system, or component.
- 2. a comprehensive list of references that support the DAEC's functional requirements and controlling (bounding) parameters.

<u>Emergency Response Data System (GL 89-15)</u>

The Emergency Response Data System (ERDS) is being established to provide data from the DAEC to the NRC through a direct link with the plant's computers. ERDS will permit a direct, real-time transfer of data from the DAEC plant computers to the NRC Operations Center. The system will be designed to be activated at the plant during an emergency which has been classified at an ALERT or higher level. The NRC portion of ERDS will receive, sort and file the data. The users will include the NRC Operations Center, the NRC Region III Office, and, if requested, the State of Iowa. The currently-installed Emergency Notification System will be used to supplement ERDS data.

Attachment 5 NG-91-0966 May 3, 1991 Page 5 of 11

Shielding Study for Revised Hydrogen Water Chemistry Program

This project's goal is to determine the feasibility of increasing hydrogen injection rates into the primary system coolant. A demonstration test was performed which determined the amount of additional hydrogen which would have to be injected into the primary coolant system to protect certain reactor vessel internals from the effects of Intergranular Stress Corrosion Cracking (IGSCC). The results of this demonstration test indicated the need for additional plant shielding due to the increased radiation levels from the additional hydrogen injection. This study will:

- 1. Determine the additional shielding requirements at various injection rates, such that the zone limits per UFSAR 12.3.1 are met, and perform a cost estimate of the design and construction costs associated with the additional shielding requirements.
- 2. Evaluate plant radiation levels due to the effects of component shielding as well as bulk shielding.
- 3. Determine the effects of additional hydrogen injection at the site boundary via skyshine measurements.

Hydrogen Water Chemistry Oxygen/Hydrogen Generator

The oxygen/hydrogen generators main purpose will be to produce hydrogen for the Hydrogen Water Chemistry Program at the plant. Producing hydrogen on-site will reduce the amount of hydrogen which is presently being delivered to the site. Development of this project is dependent upon the results of the Shielding Study for Revised Hydrogen Water Chemistry. If the decision is made to increase the hydrogen injection rates, installation of an oxygen/hydrogen generator will be considered.

Plant Life Extension

The Plant Life Extension (PLEX) project at the DAEC is aimed at obtaining an extended operating life for the plant. To accomplish this goal, the project will identify those Systems, Structures and Components (SSC) susceptible to age-related degradation and develop strategies to counter those effects. The project consists of three phases: 1) Planning development of a project plan and screening for SSC that warrant detailed life extension analyses; 2) Evaluation - detailed analyses of SSC subject to age-related degradation and implementation of aging management programs; 3) Application - preparation of the NRC submittal for license renewal.

The DAEC PLEX program is in Phase 1. We plan to issue the DAEC PLEX Program Plan and initiate a material sampling program to assess the condition of age susceptible SSC and screen those SSC for the Evaluation Phase.

Attachment 5 NG-91-0966 May 3, 1991 Page 6 of 11

Power Systems Analysis

The project scope consists of the DAEC's safety and non-safety-related AC and DC electrical distribution systems. The project scope overview includes:

- Performing an initial AC power distribution system analysis, performing a DC power distribution system analysis and implementing the Emergency Diesel Generator (EDG) transient analysis. A computerized model of the plant's electrical distribution system will be developed from these analyses. The initial model will focus on the main electrical distribution system and major loads.
- 2. Implementing short-term programmatic controls to maintain the electrical equipment data base developed in item 1 until the full program is completed.
- 3. Developing a new data base on the Computerized History and Maintenance Planning System (CHAMPS) which combines the existing data with the electrical equipment data base generated in items 1 and 2. The new data base will be called the Site Power Utilization Records (SPUR). Implementing the development of the required SPUR software.
- Expanding the computer model to include the remaining lower-voltage AC circuits, motor overloads, and breaker and fuse coordination schemes.
- 5. Implementing the long term programmatic controls, and initiating the replacement of existing plant documents with the SPUR data base.

<u>Severe Accident Management</u>

We are currently following industry and regulatory developments for direction in outlining a Severe Accident Management Program. We recognize that the Individual Plant Examination (IPE) will provide guidance in developing a program to manage severe accidents, <u>i.e.</u>, those beyond the plant's design basis.

Consequently, this program is dependant upon completion of the IPE, as well as other industry initiatives such as those being done by EPRI, NUMARC, etc.

<u>Technical Specification Improvement Program</u>

Short-Term Enhancements

The short-term enhancements of the Technical Specifications Program is divided into two phases that consists of preparation of Technical Specification (TS) amendment requests derived from a combination of the NRC line-item Improvement in Technical Specification Generic Letters (GL) and Iowa Electric-identified improvements. The identified changes for Phase 2 are as follows.

Attachment 5 NG-91-0966 May 3, 1991 Page 7 of 11

Phase 2:

GL 89-01: Removal of Radiological Effluent TS (RETS)

RTS-186:

Reverification and reformatting of Instrumentation Tables and Implementation of the BWR Owners' Group topical reports on extending surveillance intervals and allowed outage time for RPS, ECCS, Control Rod Block and PCIS instrumentation.

RTS-218:

Elimination of the Primary Containment Isolation signal on Main Steamline High Radiation. The justification for this change has been submitted as a Licensing Topical Report by the BWR Owners' Group and is currently under review by the NRC staff.

RTS-232:

Increase the allowable leakage rate limit for the Main Steamline Isolation Valves (MSIV) during the performance of 10 CFR Part 50, Appendix J testing and eliminate the requirement for the MSIV-Leakage Control System. The justification for this change has been submitted as a Licensing Topical Report by the BWR Owners' Group and is currently under review by the NRC staff.

Long-Term Improvements

The long-term program consists of a comprehensive program to rewrite TS based upon the NRC Interim Policy Statement on TS Improvements. The program will utilize the guidance from the final NRC-approved version of the BWR Owners' Group Improved TS topical report. IELP's decision to commit to this long-term improvement program is dependent upon our review of the final version of the recently-issued draft BWR/4TS (NUREG-1344).

<u>Ultrasonic Examination of Reactor Vessel Beltline Region Welds</u>

Feasibility Study, Position Paper and Drawing Development

Phase 2: The Feasibility Study, Position Paper and Drawing Development has been initiated. We are reviewing Reactor Pressure. Vessel (RPV) records, determining governing requirements and regulations (required by ASME Section XI 1989 Edition and Proposed Rule 10 CFR 50.55(a)), and developing an examination program in preparation for conducting a 10 year ISI examination of the RPV. Phase 3: Performance of Vessel Examination.

Attachment 5 NG-91-0966 May 3, 1991 Page 8 of 11

<u>Telemetry for Emergency Sirens</u>

The addition of telemetry to the Emergency Planning Public Notification System sirens will provide several benefits and increase the level of system control. Some of the benefits are:

1. The status of the siren system at any time.

2. Determination that a particular siren has been activated.

3. Identity of a loss of AC power to a system sector.

In addition, this system will assist us in minimizing or preventing problems similar to those described in NRC IN 90-34.

<u>Comprehensive Procurement Initiative</u>

The purpose of implementing this program is to minimize the impact of fraudulent activities by an equipment supplier and to provide long-term resolution to plant and NRC concerns relating to procurement activities. In order to meet these objectives we are developing and implementing a program that encompasses the NUMARC and GL 91-05 guidelines.

Evaluation

Our current procurement program will be evaluated to identify where improvements can be made by implementing the NUMARC and GL 91-05 guidelines.

Implementation

Upon completion of the Evaluation phase, we will implement the recommendations into our procurement process and procedures.

Service Water System Enhancements

<u>River Water Supply Pumps</u>

The purpose of this project is to procure a replacement River Water Pump to allow on-line rebuilding of the River Water Pumps.

The spare pump will allow for systematic refurbishment of the four installed pumps on an individual basis. For example: the spare pump will replace one of the installed pumps; the pump that was replaced by the spare will be refurbished; the refurbished pump will then replace the next installed pump requiring refurbishment; this sequence of events will continue until all pumps including the spare are refurbished. This method of refurbishment will minimize the time a pump is removed from service.

Attachment 5 NG-91-0966 May 3, 1991 Page 9 of 11

River Sediment Management System

The purpose of the project is to reduce the maintenance costs associated with sediment intrusion into the service water systems by installing flow-direction vanes (Iowa Vanes) and a retaining wall in the Cedar River at our intake structure. Completion of this project is dependent on the water level of the Cedar River, <u>i.e.</u>, the river water level has to be low enough for installation of the vanes. This project has experienced recent delays because of inclement weather, in particular, harsh winter conditions delayed construction, early spring rainfall has resulted in river water levels sufficiently high to prevent construction and unforeseen design issues.

Replace Two Electro-Hydraulic Control (EHC) Pumps

Replacement parts for the current pumps are becoming increasingly difficult to obtain. Therefore, we are considering replacing the pumps with those from another vendor for which replacement parts and pumps are readily available.

<u>Configuration Management Plan</u>

The purpose of this project is to design and implement a program to achieve comprehensive control of the updating and maintenance of plant documents. The number of changes in regulations, design, and procedures that occur for a nuclear plant over a period of time presents a difficult task to ensure that all affected procedures, drawings, equipment specifications and other controlled documents are updated to reflect the change. As part of this project a configuration management program plan is to be developed that will serve as a guide for document control and document change management. Digital imaging technology will be used to improve our ability to control the management of plant documents. This technology will be used to provide control, access and timely revision processes for documents. Initial efforts are focused on placing the plant drawings on this technology platform. The project has been divided into two major objectives: Program Plan and Digital Imaging.

Scram Frequency Reduction

The purpose of this project is to examine and implement improvements to lower the scram rate of our plant. In 1989, we formed a Scram Frequency Reduction group to review plant operating experience and industry reports to identify improvements in equipment, communications, procedures, and personnel training that can be made to reduce our scram rate in order to be consistent with industry goals. Examples of improvements that have been implemented are the conversion of some turbine trip logic from single incidence to coincident logic (e.g. two-out-of-three logic) and installation of solenoid failure detection for MSIVs and turbine master trip solenoids. Additionally, the following modifications are planned.

Attachment 5 NG-91-0966 May 3, 1991 Page 10 of 11

Mechanical Feedwater Modifications

This modification involves installation of air accumulators on feedwater regulating valves and the modification of the feedwater pump recirculation valves to fail "as is" upon a loss of air to reduce reactor scrams due plant air transients. Also, the air system isolation/control valves are being evaluated for failure modes which result in reactor scrams in order to determine if modifications can be made to prevent reactor scrams.

Digital Feedwater Control

The purpose of this project is to upgrade the present feedwater control system to a new digital control system or modify the existing control system to be an "intermediate valve selection" logic. Digital control will allow us to incorporate new technological improvements in instrumentation and computerized system control, thereby improving the system's reliability.

Turbine Electro-Hydraulic Control (EHC) System Improvements

A review of DAEC scram history (1979 - 1990) shows that over 30% of the reactor scrams have been initiated by the turbine/generator systems. The purpose of these modifications is to reduce reactor scrams due single failure within the EHC system electronics.

Long-term Instrument and Control Strategy

Analog Trip System Program Study

The purpose of this project is to determine whether installation of an analog trip system will provide an effective way to reduce scrams, particularly those which occur during Surveillance Testing. The installation of an Analog Trip System will require a substantial amount of resources; therefore, we will be closely examining industry reports and plant operating experience regarding this subject. The conclusion of this study is dependent upon the experience gained from installing the digital feedwater control system.

Metrology Lab

The purpose of this project is to establish a laboratory for calibrating our measuring and test equipment (M&TE) with control over the ambient conditions, such as temperature and relative humidity. Currently, the calibration of M&TE is performed in areas where it is difficult to repeat calibration results on a regular basis, due to less than ideal control of temperature and relative humidity.

The project has been divided into two phases which are as follows:

Phase 1: Construction of Lab Facility

Phase 2: Equipment Installation and Lab Operational

Attachment 5 NG-91-0966 May 3, 1991 Page 11 of 11

Facility Upgrade Study

This study will be conducted to determine the alternatives for increasing on-site facility space for plant, engineering and support staff.

<u>Self-Initiated Instrument & Control System Functional Inspection</u> (I&CSFI)

A self-initiated I&CSFI, using inspection techniques similar to the NRC I&CSFI inspection module, will be conducted. This inspection involves reviewing the design and implementation of a plant Instrument and Control System, which has yet to be determined, and the adequacy of associated engineering and technical support.

<u>Computer Software Quality Assurance</u>

The purpose of this program is to ensure the quality and integrity of all computer software which can impact safety-related systems, components, and structures at the Duane Arnold energy Center. Specifically, the program objectives are: defining specific requirements and standards for the development and use of software; providing the information and guidance necessary to identify, evaluate, and classify all software; establishing the method and requirements for maintaining the configuration of all nuclear-related software systems; instilling a high degree of confidence in the integrity and reliability of software systems utilized at IELP; minimizing software maintenance costs and ensuring that the use of software does not adversely affect the safe and reliable operation of the DAEC.

Maintenance_Self-Assessment

This project will be performed in accordance with NUMARC initiative guidelines. The project involves a series of audits, evaluations, and reviews aimed at providing integrated improvements to the maintenance process.