

Iowa Electric Light and Power Company

June 29, 1990

NG-90-1477

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Subject: Duane Arnold Energy Center
Docket No: 50-331
Op. License No: DPR-49
Status of Implementation of Generic Safety Issues
Reference: Request for Information on the Status of Licensee
Implementation of Generic Safety Issues Resolved
With Imposition of Requirements or Corrective
Actions (Generic Letter 90-04)
File: A-107c, A-101b

Dear Dr. Murley:

Please find attached the information requested in the referenced Generic Letter. We hope this information will assist you in updating the current implementation status of the generic safety issues (GSIs) identified herein.

The information requested is provided on the form attached to the Generic Letter. For issues that were resolved through multiple submittals, we have listed the most recent document applicable to the referenced GSI. Additional comments are provided in the "Notes" to the status list.

Should you require additional information, please contact this office.

Very truly yours,



Daniel L. Mineck
Manager, Nuclear Division

DLM/PMB/pjv+

Attachment: Status of Licensee Implementation of Generic Safety Issues

cc: P. Bessette
L. Liu
L. Root
R. McGaughy
J. R. Hall (NRC-NRR)
A. Bert Davis (Region III)
NRC Resident Office
Commitment Control # 900142

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FACILITY NAME: DAEC
 DOCKET NO.: 50-331
 LICENSEE: IELP

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	C	NRC GL 86-01 01/03/86
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	C	NG-83-3408 10/18/83
43 (B107)	Reliability Of Air Systems	All Plants	I	NG-89-0347 ⁽¹⁾ 02/21/89
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	NG-90-0200 ⁽²⁾ 01/29/90
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I	NG-90-0626 ⁽³⁾ 03/19/90
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	NG-84-0825 ⁽⁴⁾ 02/29/84
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	I	NG-90-1073 ⁽⁵⁾ 05/03/90

*Please follow attached guidance for completing this column.

**The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	NG-84-5741 ⁽⁶⁾ 12/26/84
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	NG-89-1705 06/15/89
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E	NRC GL 90-03 ⁽⁷⁾
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	NG-84-0825 02/29/84 ⁽⁸⁾ NG-85-1838 04/30/85
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	NG-84-0825 ⁽⁹⁾ 02/29/84
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	NG-86-0196 01/29/86 ⁽¹⁰⁾ NG-84-0825 02/29/84
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	NG-84-0825 02/29/84 ⁽⁹⁾
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	N/A	GL 83-28 States Item 4.1 is applicable to PWR Licensees and OL applicants only

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability- Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	N/A	
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	N/A	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	N/A	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	N/A	

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	NG-84-0825 02/29/84 ⁽⁸⁾
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	NG-85-1838 04/30/85 ⁽¹¹⁾
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	C	NG-84-2523 06/18/84 ⁽¹²⁾
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	N/A	
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	N/A	
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	N/A	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	LDR-81-215 ⁽¹³⁾ 07/07/81

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	LDR-81-215 ⁽¹³⁾ 07/07/81
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	N/A	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C	LDR-81-115 ⁽¹⁴⁾ 04/06/81
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	N/A	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	N/A	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	LDR-80-090 03/19/80

NOTES TO GENERIC SAFETY ISSUE
IMPLEMENTATION STATUS LIST

1. Generic Letter 88-14 Instrument Air Testing Program is to be completed prior to Cycle 11 startup (est. 09/90).
2. The initial actions performed in accordance with the recommendations of Generic Letter 89-13 are to be completed prior to Cycle 11 startup (est. 09/90).
3. The hydrogen and oxygen instrument ranges are to be revised prior to Cycle 11 startup (est. 09/90). The NRC Safety Evaluation Report (SER) for conformance to R.G. 1.97 was issued to Iowa Electric on 05/09/90.
4. The NRC SER to Salem ATWS Item 1.1 was issued to Iowa Electric on 05/06/85.
5. A Containment Isolation Monitoring System is to be installed and operable prior to Cycle 11 startup (est. 09/90).
6. The NRC SER for Salem ATWS Item 2.1 (Subpart 1) was issued to Iowa Electric on 06/23/86. The SER states that the Vendor Interface portion of this item (Subpart 2) is not resolved. However, no Licensee actions are outstanding.
7. Iowa Electric's response to Generic Letter 90-03 is due on 07/26/90.
8. The NRC SER for Salem ATWS Items 3.1.1, 3.1.2, and 4.5.1 was issued to Iowa Electric on 08/06/85.
9. The NRC SER for Salem ATWS Items 3.1.3 and 3.2.3 was issued to Iowa Electric on 07/16/85.
10. The NRC SER for Salem ATWS Items 3.2.1 and 3.2.2 was issued to Iowa Electric on 05/29/86.
11. The NRC SER for Salem ATWS Items 4.5.2 and 4.5.3 were issued to Iowa Electric on 05/31/89 and 06/02/89 respectively.
12. Inspections discussed in NG-84-2523 were completed by 06/85. NRC GL 88-01 is assumed to be beyond the scope of the issue.
13. Technical Specification Amendment 71 was approved on 08/81.
14. The NRC SER for "Adequacy of Offsite Power Systems" was issued to Iowa Electric on 12/31/81.