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 MURLEY,T.E. Office of Nuclear Reactor Regulation, Director (Post 870411

SUBJECT: Forwards info requested in Generic Ltr 89-21 re status of implementation of USI requirements.

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Iowa Electric Light and Power Company

December 21, 1989

NG-89-3594

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
MAIL STATION P1-137
Washington, DC 20555

Subject: Duane Arnold Energy Center
Docket No: 50-331
Op. License No: DPR-49
Status of Implementation of Unresolved
Safety Issue Requirements
Reference: Request for Information Concerning Status
of Implementation of Unresolved Safety
Issue (USI) Requirements (Generic
Letter 89-21)
File: A-107a, A-105

Dear Dr. Murley:

Please find attached the information requested in the referenced Generic Letter. We hope this information will assist you in updating the status of Unresolved Safety Issue (USI) resolution at the Duane Arnold Energy Center.

The information requested has been added to the USI status list provided with the Generic Letter. We have attempted to provide dates that correspond to when the work was actually completed or, in the cases of license amendments or regulatory exemptions, the actual implementation dates.

This response is provided on December 21, 1989, consistent with our discussion with the NRC Project Manager for the Duane Arnold Energy Center.

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PDR ADOCK 05000331
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Dr. Thomas E. Murley
December 21, 1989
NG-89-3594
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Should you require additional information, please contact this office.

Very truly yours,



Daniel L. Mineck
Manager, Nuclear Division

DLM/PLB/pjv+

Attachment: Unresolved Safety Issues For Which A Final Technical Resolution
Has Been Achieved

cc: P. Bell
L. Liu
L. Root
R. McGaughy
J. R. Hall (NRC-NRR)
A. Bert Davis (Region III)
NRC Resident Office
Commitment Control Nos. 890384, 890208, 890404, 880169, 890382

ENCLOSURE 1

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

| <u>USI/MPA NUMBER</u> | <u>TITLE</u> | <u>REF. DOCUMENT</u> | <u>APPLICABILITY</u> | <u>STATUS/DATE*</u> | <u>REMARKS</u> |
|-----------------------|--|--|----------------------|---------------------|----------------|
| A-1 | Water Hammer | SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions | A11 | NC | |
| A-2/ MPA D-10 | Asymmetric Blowdown Loads on Reactor Primary Coolant Systems | NUREG-0609 GL 84-04, GDC-4 | PWR | NA | |
| A-3 | Westinghouse Steam Generator Tube Integrity | NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements) | W-PWR | NA | |
| A-4 | CE Steam Generator Tube Integrity | NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements) | CE-PWR | NA | |
| A-5 | B&W Steam Generator Tube Integrity | NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements) | B&W-PWR | NA | |
| E A-6 | Mark I Containment Short-Term Program | NUREG-0408 | Mark I-BWR | C: 10/78 | |

- * C - COMPLETE
NC - NO CHANGES NECESSARY
NA - NOT APPLICABLE
I - INCOMPLETE
E - EVALUATING ACTIONS REQUIRED

| <u>USI/MPA NUMBER</u> | <u>TITLE</u> | <u>REF. DOCUMENT</u> | <u>APPLICABILITY</u> | <u>STATUS/DATE*</u> | <u>REMARKS</u> |
|-----------------------|---|--|----------------------|---------------------|----------------|
| A-7/ D-01 | Mark I Long-Term Program | NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57 | Mark I-BWR | C: 9/85 | |
| A-8 | Mark II Containment Pool Dynamic Loads | NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16 | Mark II-BWR | NA | |
| A-9 | Anticipated Transients Without Scram | NUREG-0460, Vol. 4 10 CFR 50.62 | A11 | C: 7/88 | |
| A-10/ MPA B-25 | BWR Feedwater Nozzle Cracking | NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11 | BWR | C: 12/81 | |
| A-11 | Reactor Vessel Material Toughness | NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26 | A11 | C: 5/85 | |
| A-12 | Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports | NUREG-0577, Rev. 1 SRP Revision 5.3.4 | RWR | NA | |
| A-17 | Systems Interactions | Ltr: DeYoung to licensees - 9/72 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements) | A11 | NA | |
| A-24/ MPA B-60 | Qualification of Class 1E Safety-Related Equipment | NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 CI 85-15 | A11 | C: 1/85 | |

| <u>USI/MPA NUMBER</u> | <u>TITLE</u> | <u>REF. DOCUMENT</u> | <u>APPLICABILITY</u> | <u>STATUS/DATE*</u> | <u>REMARKS</u> |
|------------------------|---|---|-------------------------|---------------------|--|
| A-26/ MPA B-04 | Reactor Vessel Pressure Transient Protection | DOR Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11 | PWR. | NA | |
| A-31 | Residual Heat Removal Shutdown Requirements | NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7 | All OLS After 01/79. | NA | |
| A-36/ C-10, C-15 | Control of Heavy Loads Near Spent Fuel | NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80 | All | C: 6/85 | |
| A-39 | Determination of SRV Pool Dynamic Loads and Pressure Transients | NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SPP 6.2.1.1.C | BWR | NA | Mark I Containments dealt with as part of USI A-7 |
| A-40 | Seismic Design Criteria | SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509 | All | NC | |
| A-42/ MPA B-05 | Pipe Cracks in Boiling Water Reactors | NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01 | BWR | I: 7/88 | Awaiting NRC approval of response to GL 88-01 and Technical Specification change |

| <u>USI/MPA NUMBER</u> | <u>TITLE</u> | <u>REF. DOCUMENT</u> | <u>APPLICABILITY</u> | <u>STATUS/DATE*</u> | <u>REMARKS</u> |
|-----------------------|---|---|--|-----------------------------|---|
| A-43 | Containment Emergency Sump Performance | NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.82 (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements | A11 | NC | |
| A-44 | Station Blackout | RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63 | A11 | I: Schedule not yet certain | Schedule due within 30 days of the notification provided under 10CFR50.63(c)(3) |
| A-45 | Shutdown Decay Heat Removal Requirements | SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements) | A11 | I: 8/92 | Will address as part of IPE. Any hardware change will follow. |
| A-46 | Seismic Qualification of Equipment in Operating Plants | NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03 | A11 | I: 9/91 | |
| A-47 | Safety Implication of Control Systems | NUREG-1217, NUREG-1218 GL 89-19 | A11 | E: 3/90 | Response due to NRC on GL 89-19 by 3/20/90 |
| A-48 | Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment | 10 CFR 50.44 SECY 89-122 | A11, except PWRs with large dry containments | NC | |
| A-49 | Pressurized Thermal Shock | RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11 | PWR | NA | |