

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8706300604 DDC DATE: 87/06/23 NOTARIZED: NO DOCKET #
 FACIL: 50-331 Duane Arnold Energy Center, Iowa Electric Light & Pow 05000331
 AUTH. NAME AUTHOR AFFILIATION
 THORSTEINSON, J. Iowa Electric Light & Power Co.
 HANNEN, R. L. Iowa Electric Light & Power Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-019-00: on 870617, reactor protection sys actuation occurred. Caused by procedural inadequacy. Reactor protection sys trip reset & surveillance test suspended used as immediate corrective actions. W/870623 ltr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD3-1 LA	1 1	PD3-1 PD	1 1
	CAPPUCCI, A	1 1		
INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	AEOD/DOA	1 1	AEOD/DSP/ROAB	2 2
	AEOD/DSP/TPAB	1 1	DEDRO	1 1
	NRR/DEST/ADE	1 0	NRR/DEST/ADS	1 0
	NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
	NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
	NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
	NRR/DEST/RSE	1 1	NRR/DEST/SGB	1 1
	NRR/DLPQ/HFB	1 1	NRR/DLPQ/GAB	1 1
	NRR/DOEA/EAB	1 1	NRR/DREP/RAB	1 1
	NRR/DREP/RPB	2 2	NRR/PMAS/ILRB	1 1
	NRR/PMAS/PTSB	1 1	REG FILE 02	1 1
	RES DEPY GI	1 1	RGNS FILE 01	1 1
EXTERNAL:	EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
	NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1

TOTAL NUMBER OF COPIES REQUIRED: LTTR 42 ENCL 40

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Duane Arnold Energy Center (DAEC)	DOCKET NUMBER (2) 0 5 0 0 0 3 3 1 1	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Protection System Actuation Due to Procedural Error

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
									None		0 5 0 0 0
0 6	1 7	8 7	8 7	0 1 9	0 0	0 6	2 3	8 7			0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

OPERATING MODE (9) N	<input type="checkbox"/> 20.402(b)	<input checked="" type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)		

POWER LEVEL (10) **0 1 0 1 0**

LICENSEE CONTACT FOR THIS LER (12)

NAME Jeffrey C. Thorsteinson, Technical Support Supervisor	TELEPHONE NUMBER
	AREA CODE 3 1 9 8 5 1 1 - 7 2 3 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) | NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 17, 1987, at 0009 hours, a Reactor Protection System (RPS) actuation occurred due to Main Steam Line High Radiation signals generated during augmented surveillance testing on logic systems. The plant was shutdown at the time and no rod motion occurred.

As immediate corrective actions, the RPS trip was reset (at 0011 hours) and the surveillance test suspended. The test was modified and was completed without further incident at approximately 0330 hours.

The root cause of the event was a procedural inadequacy since the procedure did not include a step to reset the RPS trip on one division (half scram) prior to beginning testing on the other RPS division.

All systems operated as designed and, as a result, there was no threat to the health and safety of the public. This event is being reported pursuant to 10 CFR 50.73.a.2.iv.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Duane Arnold Energy Center (DAEC)	DOCKET NUMBER (2) 0 5 0 0 0 3 3 1	LER NUMBER (E)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 7	— 0 1 9	— 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 388A's) (17)

On June 17, 1987, at 0009 hours, a Reactor Protection System (RPS, EIIS System Code JC) actuation occurred due to Main Steam Line High Radiation signals generated during augmented surveillance testing on logic systems. The plant was shutdown at the time and no rod motion occurred.

As immediate corrective actions, the RPS trip was reset (at 0011 hours) and the surveillance test suspended. The test was modified and was completed without further incident at approximately 0330 hours. An investigation into the circumstances revealed many factors contributing to the cause of this event. These factors included the length of the test, the complexity of the test, and the fact that the test was being performed for the first time. These factors combined to make it difficult for the operators and the technicians to recognize a procedural inadequacy.

The root cause of the event was a procedural inadequacy since the procedure did not include a step to reset the RPS trip on one division (half scram) prior to beginning testing on the other RPS division.

This event is considered unique in that it involves a new procedure that was being performed to comply with augmented testing commitments. This situation does not occur in the surveillances normally used at the DAEC. The procedure had received independent review, but the complexity of the procedure contributed to the inability of reviews to recognize that a required step was missing. While there have been unexpected actuations of the RPS previously at the DAEC, they are not considered similar to this actuation due to the unique circumstances involved with this event. No further corrective actions are warranted.

All systems operated as designed and, as a result, there was no threat to the health and safety of the public. This event is being reported pursuant to 10 CFR 50.73.a.2.iv.

Iowa Electric Light and Power Company

June 23, 1987
DAEC-87-0640

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Subject: Duane Arnold Energy Center
Docket No. 50-331
Op. License DPR-49
Licensee Event Report No. 87-019

Gentlemen:

In accordance with 10 CFR 50.73 please find attached a copy of the subject Licensee Event Report.

Very truly yours,

 6/23/87

Rick L. Hannen
Plant Superintendent - Nuclear

RLH/JCT/go

Attachment - LER 87-019

cc: Mr. A. Bert Davis
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

NRC Resident Inspector - DAEC

File A-118a

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11