

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8105040345 DOC. DATE: 81/04/28 NOTARIZED: NO DOCKET #
 FACIL: 50-331 Duane Arnold Energy Center, Iowa Electric Light & Pow 05000331
 AUTH. NAME AUTHOR AFFILIATION
 RAGER, M.S. Iowa Electric Light & Power Co.
 RECIP. NAME RECIPIENT AFFILIATION
 Region 3, Chicago, Office of the Director

SUBJECT: LER 77-057/03X-1: on 770714, emergency syc water pump A was inoperable. Caused by river temp exceeding temp that allowed pump to provide sufficient flow for design cooling tasks. Pump replaced.

DISTRIBUTION CODE: A002S COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 1+1
 TITLE: Incident Reports

NOTES:

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INTERNAL:	A/D COMP&STRU06	1	1	A/D ENV TECH 07	1	1
	A/D MATL & QU08	1	1	A/D OP REACT009	1	1
	A/D PLANT SYS10	1	1	A/D RAD PROT 11	1	1
	A/D SFTY ASSE12	1	1	ACC EVAL BR 14	1	1
	AEOD	3	3	AEOD/DMU	3	3
	ASLBP/J. HARD	1	1	AUX SYS BR 15	1	1
	CHEM ENG BR 16	1	1	CONT SYS BR 17	1	1
	CORE PERF BR 18	1	1	DIR, ENGINEERI20	1	1
	DIR, HUM FAC S21	1	1	DIR, SYS INTEG22	1	1
	EFF TR SYS BR23	1	1	EQUIP QUAL BR25	1	1
	GEOSCIENCES 26	1	1	I&C SYS BR 29	1	1
	I&E 05	1	1	JORDAN, E./IE	1	1
	LIC GUID BR 30	1	1	MATL ENG BR 32	1	1
	MECH ENG BR 33	1	1	NRC PDR 02	1	1
	OR ASSESS BR 35	1	1	POWER SYS BR 36	1	1
	RAD ASSESS BR39	1	1	REACT SYS BR 40	1	1
	<u>REG FILE</u> 01	1	1	REL & RISK A 41	1	1
	SFTY PROG EVA42	1	1	STRUCT ENG BR44	1	1
EXTERNAL:	ACRS 46	16	16	LPDR 03	1	1
	NSIC 05	1	1			

MAY 05 1981

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Iowa Electric Light and Power Company

April 28, 1981
DAEC-81-268

Mr. James G. Keppler, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission - Region III
799 Roosevelt Road
Glen Ellyn, IL 60137

Subject: Licensee Event Report No. 77-057 UPDATE REPORT:
(30 day) Previous Report
Date 8-5-77
File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49,
Technical Specifications and Bases for Duane Arnold Energy Center
and Regulatory Guide 10.1, please find attached a copy of the
subject Licensee Event Report. (Total of 3 copies transmitted).

Very truly yours,

Daniel L. Mineck
Chief Engineer
Duane Arnold Energy Center

DLM/MSR/pl

Docket 50-331

attachment

cc: Director, Office of Inspection and Enforcement (30)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Director, Management Information and Program Control (3)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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U. S. Nuclear Regulatory Commission
c/o Document Management Branch
Washington, D. C. 20555

NRC Resident Inspector - DAEC

LICENSEE EVENT REPORT

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7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CON'T
0 1 | R | E | P | O | R | T | S | O | U | R | C | E | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 3 | 1 | 7 | 0 | 7 | 1 | 1 | 4 | 7 | 7 | 8 | 0 | 4 | 2 | 8 | 8 | 1 | 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35
REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During normal power operation, the river temperature was above 89.9 deg
0 3 | rees F for 4 hours and 20 minutes. The "A" Emergency Service Water pump
0 4 | -1P-99A-could provide sufficient flow to meet its design cooling tasks
0 5 | with a river water temp of 89.9 degrees F or less. The "A" loop was the
0 6 | before inop until the water temp fell below this temp. This initiated a
0 7 | 7-day LCO in accordance with Tech Spec 3.8.C.2. The "B" FSW System was
0 8 | proven operable. Was a repetitive problem.

0 9 | S | H | 11 | B | 12 | Z | 13 | P | U | M | P | X | X | 14 | Z | 15 | Z | 16
7 8 9 10 11 12 13 14 15 16 17 18 19 20
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
17 | L | E | R | / | R | O | R | E | P | O | R | T | N | U | M | B | E | R | 7 | 7 | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 1 | 32
EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
18 | C | 18 | Z | 19 | Z | 20 | Z | 21 | 0 | 0 | 0 | 0 | 0 | 22 | N | 23 | N | 24 | A | 25 | L | 0 | 9 | 1 | 5 | 26
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | Initial pump design was inadequate to provide proper flow to fulfill coo
1 1 | ling requirements. The Layne and Bowler type 12RH pumps were replaced du
1 2 | ring the 1980 refuel outage with a Johnston Pump, type 12DC. These pumps
1 3 | are expected to fulfill the flow requirements necessary for adequate coo
1 4 | ling. No further corrective action is planned.

1 5 | E | 28 | 0 | 8 | 8 | 29 | NA | 30 | A | 31 | Operator Observation
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION

1 6 | Z | 33 | Z | 34 | NA | 35 | NA | 36
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE

1 7 | 0 | 0 | 0 | 37 | Z | 38 | NA | 39
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION

1 8 | 0 | 0 | 0 | 40 | NA | 41
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
PERSONNEL INJURIES NUMBER DESCRIPTION

1 9 | 7 | 42 | NA | 43
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION

2 0 | N | 44 | NA | 45
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
PUBLICITY ISSUED DESCRIPTION

NAME OF PREPARER Matthew S. Rager

PHONE: 319-851-5611

NRC USE ONLY

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