

Iowa Electric Light and Power Company

July 22, 1980
LDR-80-203

LARRY D. ROOT
ASSISTANT VICE PRESIDENT
NUCLEAR GENERATION

Mr. James G. Keppler
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Re: Duane Arnold Energy Center

Subject: IE Bulletin 80-14, Degradation of BWR
Scram Discharge Volume Capability

File: A-101a, NRC-2, Bulletin 80-14

Dear Mr. Keppler:

In response to your letter transmitting the subject bulletin concerning the scram discharge volume (SDV) level switch and drain valve failure events which could result in a degraded SDV capability due to undetected water, we have completed the recommended licensee actions. The following discussion is provided to briefly describe the actions taken at DAEC to address these NRC IE Bulletin concerns.

BULLETIN ITEM 1. Plant maintenance records, Abnormal Occurrence (AO), and Licensee Event Report (LER) files were reviewed for instances of degradation of any SDV level switch which was or may have been caused by a damaged or bent float assembly. One occurrence (AO Report 75-020) was identified during surveillance testing on April 18, 1975 which involved a SDV level switch with a bent float stem at DAEC. The cause of this occurrence was attributed to a manufacturing problem since a plastic foreign object was found in the switch reservoir. Subsequent surveillance testing revealed no further problems with the operability of this level switch. No other float stem problems have been identified with any of the DAEC SDV level switches.

AO 75-020

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BULLETIN ITEM 2. Plant records were reviewed for instances of degradation of SDV vent or drain valve operability. No such problems were identified prior to the recent DAEC reactor scram testing performed in accordance with IE Bulletin 80-17. The recent scram testing revealed a problem with SDV drain valve leakage which has been subsequently corrected (Refer to LER 80-028 for detailed explanation).

The SDV vent and drain valve closure times were measured as part of the recently performed reactor scram tests. The average SDV vent valve (CV-1859) closure time was 23.8 seconds and the average SDV drain valve (CV-1867) closure time was 21.4 seconds. There are no DAEC Technical Specification or other requirements for acceptable SDV vent and drain valve closure times.

BULLETIN ITEM 3. DAEC has prepared a surveillance test procedure (BS-6, Scram Discharge Volume and Piping Water Test) which will verify by weekly testing that the SDV vent and drain valves are operable. These valves are maintained in the open position as evidenced by valve position indication lights in the control room and by verification by procedure (BS-6) on a daily basis that these valves are open.

In order to comply with the 24 hour prompt notification requirement, an OPERATING ORDER (#2-94) has been issued requiring the Shift Supervising Engineer notify the NRC Operations Center in Bethesda, Md. via the emergency notification system "red phone" if these valves are not operable or are closed for more than 1 hour in any 24 hour period during operation.

BULLETIN ITEM 4. No instances of water hammer or damage which may have been caused by water hammer in SDV related piping have been identified by review of plant records at DAEC.

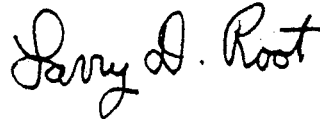
BULLETIN ITEM 5. A review of the DAEC Technical Specification and surveillance test procedures (STP) was performed to ensure that degradation of any SDV level switch due to a damaged float or other cause would be detected by periodic testing. The DAEC SDV level switches are functionally tested quarterly and are calibrated during each refueling outage (STP 41A006, Discharge Volume High Water Level Instrument Functional Test/Calibration).

Mr. James G. Keppler
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BULLETIN ITEM 6. Following the recent scram testing and associated SDV outage work, all of the DAEC SDV level switches were functionally tested. This surveillance testing (STP 41A006) which verified the satisfactory operation of these switches was completed on July 17, 1980.

If you have any questions or desire further information concerning this matter, please contact this office.

Very truly yours,



Larry D. Root
Assistant Vice President
Nuclear Generation

LDR/DWT/pl

Docket No. 50-0331

cc: U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Division of Reactor Operations Inspection
Washington, D. C. 20555

✓ U.S. Nuclear Regulatory Commission
c/o Document Management Branch
Washington, D. C. 20555

D. Tooker
D. Arnold
L. Liu
S. Tuthill
K. Meyer
D. Mineck
D. Wilson
J. Van Sickel
NRC Resident Inspector-DAEC

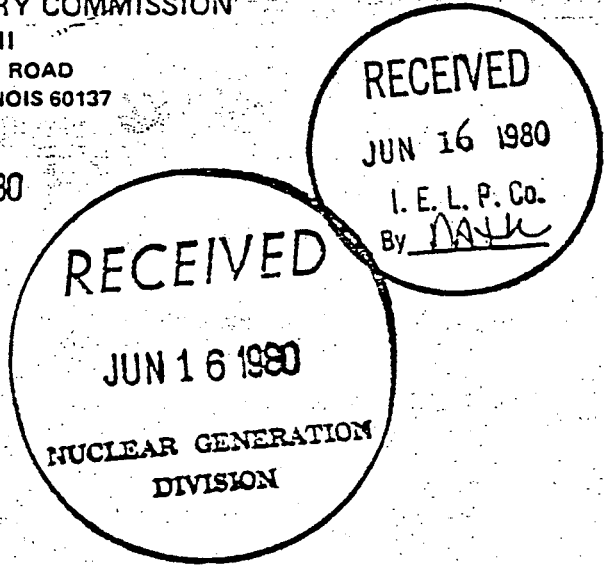


UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 REGION III
 799 ROOSEVELT ROAD
 GLEN ELLYN, ILLINOIS 60137

JUN 12 1980

Docket No. 50-331

Iowa Electric Light and Power
 Company
 ATTN: Mr. Duane Arnold
 President
 IE Towers
 P. O. Box 351
 Cedar Rapids, IA 52406



Gentlemen:

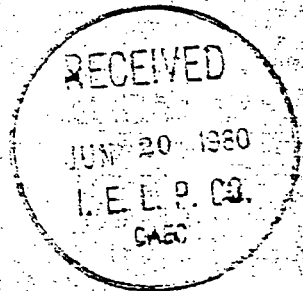
Enclosed is IE Bulletin No. 80-14 which requires action by you with regard to your power reactor facility with an operating license.

In order to assist the NRC in evaluating the value/impact of each bulletin on licensees, it would be helpful if you would provide an estimate of the manpower expended in conduct of the review and preparation of the report required by the bulletin. Please estimate separately the manpower associated with corrective actions necessary following identification of problems through the bulletin.

Should you have any questions regarding this bulletin or the actions required by you, please contact this office.

Sincerely,

James G. Keppler
 James G. Keppler
 Director



Enclosure: IE Bulletin
 No. 80-14

cc w/encl:
 D. Mineck, Chief
 Engineer
 Central Files
 Director, NRR/DPM
 Director, NRR/DOR
 PDR
 Local PDR
 NSIC
 TIC

RESPONSE DUE AUG 6, 1980

• Arnold	• Meyer	★ Mineck	• Rehrauer
• Tuthill	• Balas	• Hannen	• Blase
• Liu	• Nodan	• Kuehn	• Chen
• Root	• Salmon	• Tepy	• Ellis
• Youngs	• Ward	• Van Sickle	• Heskje
• Pitner	• Gucciardo	• York	• Shearer
• Benfield	• Loehrlein	• Young	• Shugart
• Lowenstein	• Hammond	• Wilson	• Comm. Cont.
• Nash (GE)	_____	_____	• Filis
	_____	_____	• A101a
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