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FROM: Iowa Electric Light & Power Co. Cedar Rapids, Iowa GG. Hunt		DATE OF DOC 3-21-75	DATE REC'D 4-2-75	LTR xxxx	TWX	RPT	OTHER
TO: Mr. James G. Keppler		ORIG 1-signed	CC	OTHER	SENT AEC PDR <u>xxx</u> SENT LOCAL PDR <u>xxxxx</u>		
CLASS	UNCLASS xxxxxx	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-331		

DESCRIPTION: Ltr trans the following:	ENCLOSURES: Appendix B to DAEC Startup Test Results
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PLANT NAME: Duane Arnold

FOR ACTION/INFORMATION 4-4-75 JGB

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IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office
CEDAR RAPIDS, IOWA

DUANE ARNOLD ENERGY CENTER
PALO, IOWA
MARCH 21, 1975
DAEC - 75 - 117



Mr. James G. Keppler
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

50-331

SUBJECT: DAEC Startup Testing Report

REFERENCE: Letter from G. G. Hunt to J. G. Keppler
dated 12-23-74.

FILE: A-289h

Dear Mr. Keppler:

In accord with Technical Specification 6.11.1, please find enclosed one (1) copy of Appendix B to the DAEC Startup Testing Report. This appendix is submitted to fulfill the requirement of the first three (3) month supplementary report subsequent to transmittal of the main body of the report via the above referenced letter. An additional 39 copies are being mailed under separate cover.

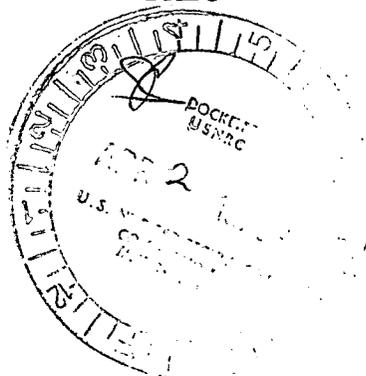
Sincerely,

A handwritten signature in black ink, appearing to read "G. G. Hunt".

G. G. Hunt
Chief Engineer
DAEC

DAM/GGH/pag
Encl.

cc: C. W. Sandford
J. A. Wallace
E. L. Hammond
B. R. York
J. H. Gebert
R. R. Rinderman
D. L. Wilson
L. D. Root
H. W. Rehrauer



3454

MAR 27 1975

APPENDIX B TO DAEC

STARTUP TEST RESULTS

DUANE ARNOLD ENERGY CENTER

UNIT 1

PREPARED BY: *D. A. Moen*
D. A. Moen
Reactor & Plant Performance Engineer

APPROVED BY: *E. L. Hammond*
E. L. Hammond
Assistant Chief Engineer

APPROVED BY: *G. G. Hunt* *3-21-75*
G. G. Hunt
Chief Engineer

1.0 INTRODUCTION

This Appendix supplements the Startup Testing Report submitted to the USAEC on December 23, 1974 by summarizing testing completed between that date and March 21, 1975. Parenthetical identifiers cited in Section 2 below refer to acceptance criteria stated in Appendix A.

2.0 SUMMARY OF TEST RESULTS

20.28 STI-28 Shutdown from Outside of the Control Room.

This test was performed on February 7, 1975 with the following result.

- (C-28.2.1) The reactor was brought to the point where cooldown could have been initiated without intervention from the Control Room. Reactor vessel pressure and level remained stable.

20.70 STI-70 Reactor Water Cleanup System.

As stated in the body of the Startup Testing Report, a repeat of the normal and hot standby modes of this test was required due to the discovery of incorrect flow orifices in the system. This testing was repeated with the following result.

- (C-70.1.1) Reactor water quality was maintained within Fuel Warranty specifications.
- (C-70.2.1) The non-regenerative heat exchanger tube side outlet temperature remained well below the 130°F limit in all modes. This limit was stated incorrectly as 140°F in Appendix A.
- (C-70.2.2) The available NPSH in hotstandby mode was 371 ft. of 520°F water; far in excess of the 10 ft. required.
- (C-70.2.3) Cooling water supplied to the non-regenerative heat exchangers was maintained within the limits established by the process diagrams.