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Iowa Electric Light and Power Company

August 14, 1992
NG-92-3713

Mr. A. Bert Davis
Regional Administrator
Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Subject: Duane Arnold Energy Center
Docket No: 50-331
Op. License DPR-49
July 1992 Monthly Operating Report

Dear Mr. Davis:

Please find enclosed the Duane Arnold Energy Center Monthly Operating Report for July 1992. The report has been prepared in accordance with the guidelines of NUREG-0020 and distribution has been made in accordance with DAEC Technical Specifications, Section 6.11.1.c.

Very truly yours,

David L. Wilson
David L. Wilson
Plant Superintendent - Nuclear

DLW/RBW/pjv
Enclosures
File A-118d

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R PDR

OPERATING DATA REPORT

DOCKET NO. 050-0331
DATE 08-14-92
COMPLETED BY Richard Woodward
TELEPHONE (319) 851-7318

OPERATING STATUS

Notes

1. Unit Name: Duane Arnold Energy Center
2. Reporting Period: July 1992
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 565 (Turbine)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515

8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since the Last Report, Give Reasons: N/A

9. Power Level to Which Restricted, If Any (Net MWe): N/A

10. Reasons for Restrictions, If Any: N/A

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	<u>744.0</u>	<u>5111.0</u>	<u>153383.0</u>
12. Number of Hours Reactor Was Critical	<u>744.0</u>	<u>3732.4</u>	<u>112520.7</u>
13. Reactor Reserve Shutdown Hours	<u>.0</u>	<u>.0</u>	<u>192.8</u>
14. Hours Generator On-Line	<u>744.0</u>	<u>3687.2</u>	<u>109595.3</u>
15. Unit Reserve Shutdown Hours	<u>.0</u>	<u>.0</u>	<u>.0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1220299.0</u>	<u>5555433.4</u>	<u>148609029.8</u>
17. Gross Electrical Energy Generated (MWH)	<u>402464.5</u>	<u>1843297.5</u>	<u>49809623.5</u>
18. Net Electrical Energy Generated (MWH)	<u>379749.8</u>	<u>1722877.5</u>	<u>46687389.6</u>
19. Unit Service Factor	<u>100.0</u>	<u>72.1</u>	<u>71.5</u>
20. Unit Availability Factor	<u>100.0</u>	<u>72.1</u>	<u>71.5</u>
21. Unit Capacity Factor (Using MDC Net)	<u>99.1</u>	<u>65.5</u>	<u>59.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>94.9</u>	<u>62.7</u>	<u>56.6</u>
23. Unit Forced Outage Rate	<u>0.0</u>	<u>0.0</u>	<u>12.7</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of each: <u>N/A</u>)			
25. If Shutdown at End of Report Period, Est. Date of Startup: <u>N/A</u>			

(9/77)

000025

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-0331
DATE 08-14-92
COMPLETED BY Richard Woodward
TELEPHONE (319) 851-7318

MONTH July 1992

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>513</u>
2	<u>514</u>
3	<u>519</u>
4	<u>487</u>
5	<u>499</u>
6	<u>515</u>
7	<u>516</u>
8	<u>503</u>
9	<u>467</u>
10	<u>515</u>
11	<u>515</u>
12	<u>510</u>
13	<u>512</u>
14	<u>519</u>
15	<u>516</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

16	<u>517</u>
17	<u>516</u>
18	<u>514</u>
19	<u>515</u>
20	<u>521</u>
21	<u>518</u>
22	<u>517</u>
23	<u>518</u>
24	<u>518</u>
25	<u>514</u>
26	<u>485</u>
27	<u>515</u>
28	<u>512</u>
29	<u>486</u>
30	<u>520</u>
31	<u>519</u>

REFUELING INFORMATION

DOCKET NO. 50-0331
DATE 08-14-92
COMPLETED BY Richard Woodward
TELEPHONE (319) 851-7318

1. Name of facility.
 - a. Duane Arnold Energy Center
2. Scheduled date for next refueling shutdown.
 - a. July, 1993
3. Scheduled date for restart following refueling.
 - a. September, 1993
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

N/A
5. Scheduled date(s) for submitting proposed licensing action and supporting information.

N/A
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

N/A
7. The number of fuel assemblies (a) in the core, (b) in the spent fuel storage pool.
 - a. 368 b. 1152
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
 - a. 2050 - Licensed Capacity or
 - b. 1898 under the presently installed storage rack capacity.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
 - a. 2000 - Licensed Capacity or
 - b. 1997 under the presently installed storage rack capacity.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: JULY 1992

Docket No.: 050-0331

Unit: Duane Arnold Energy Center

Date: 08-14-92

Completed By: Richard Woodward

Telephone: (319) 851-7318

No.	Date	Type(1)	Duration (Hours)	Reason(2)	Method of Shutting (3) Down Reactor	Licensee Event Report #	System Code (4)	Comp. Code (5)	Cause
THERE WERE NO SHUTDOWNS OR DAILY AVERAGE POWER REDUCTIONS GREATER THAN 20% DURING THE MONTH.									

1 - F: Forced
S: Scheduled

2 - Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3 - Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continued
5-Reduced Load
9-Other (Explain)

4 - Exhibit G-
Instructions for
Preparation of Data
Entry Sheets for
Licensee Event Report
(LER) File (NUREG-
0161)

5 - Exhibit 1-
Same Source

(9/77)

MAJOR/SAFETY RELATED MAINTENANCE

Docket No.: 050-0331
Unit: Duane Arnold Energy Center
DATE: 08-14-92
COMPLETED BY: Richard Woodward
Telephone: (319) 851-7318

DATE	SYSTEM	COMPONENT	DESCRIPTION
07/09/92	High Pressure Coolant Injection (HPCI) System	Steam Supply Outboard Isolation Valve M02239	Backseated the valve to fix packing leak.
07/26/92	Main Steam	Control Valve 4	CV4 would not close during test of control valve fast closure scram logic for Reactor Protection System Channels B1 & B2. Three days later after trouble-shooting the test push button, C4 closed and the RPS logic check was completed satisfactorily.

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

DOCKET NO: 050-0331
DATE: 08/14/92
COMPLETED BY: Richard Woodward
TELEPHONE: (319) 851-7318

Monthly Operational Overview for July, 1992

During July DAEC operated at 98.9% thermal and 95.7% Gross Electric Capacity Factor, sending 510.4 MWe onto the grid. Gross electric capacity factor losses were as follows:

Capacity	1.0%
Weather	1.5%
Equipment	1.8%

	4.3%

The capacity reductions were to fix a packing leak, follow load, perform a surveillance test, and adjust control rods.

Two 10 CFR 50.73 reportable events occurred during this reporting period. There were no unplanned Limiting Conditions for Operation (LCOs) associated with the Emergency Core Cooling Systems (ECCSs).

On July 7, 1992, with the plant operating at 100% power, the 'A' and 'B' Emergency Diesel Generators (EDGs) automatically started. This occurred due to a momentary grid disturbance caused by an electrical storm. Although the EDGs started, they were not required to load. In addition to the EDG starts, one of two main generator output breakers opened and the 'A' Reactor Water Cleanup (RWCU) pump tripped (as designed). The primary instrument and service air compressors were momentarily deenergized when faults were sensed on both the normal and backup feedlines supplying their transformers. The circuit breakers on these lines opened as designed. This event had no effect on the safe operation of the plant.

LER 92-0011

During March and April of 1992, when the plant was in cold shutdown during a refueling outage, testing was done on the Cable Spreading Room (CSR) carbon dioxide fire suppression system and its affect on control room habitability. Due to carbon dioxide infiltration into the control room, a temporary vent damper assembly was installed in the CSR. In June, testing determined that the required positive pressure in the control room under accident conditions could not be met if the vent damper assembly failed open. On 7-10-92, with the plant at 99.9% power, the vent damper assembly was determined to be non-seismic. There was no immediate effect on plant safety but it was recognized that operator action would be required to assure control room habitability was not degraded during a seismic event. The cause for the lack of seismic qualification and the root cause for the positive pressure test failure was the failure to recognize the need for assured closure of the vent damper given the variation in unidentified leaks in the system. A safety related and seismic modification will replace the current vent damper assembly. Temporary procedure changes provide compensatory actions until the modification is complete. Responsible personnel have been made aware of the sensitivity of the system and the unidentified leaks are being pursued.

LER 92-0012 (pending)