OPERATING DATA REPORT

DOCKET NO. 050-0331
DATE 9-15-80
COMPLETED BY J. VanSickel
TELEPHONE 319-851-5611

OPERATING STATUS

1. Unit Name: Duane Arnold Energy Center
2. Reporting Period: August, 1980
3. Licensed Thermal Power (MWt): 1658
* 4. Nameplate Rating (Gross MWe): 565 (Turbine rating)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5,855	48,935
12. Number Of Hours Reactor Was Critical	729.1	3,923	34,487
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	719	3,814.4	33,633.6
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,066,752	4,951,872	42,081,432
17. Gross Electrical Energy Generated (MWH)	348,310	1,653,809	14,078,175
18. Net Electrical Energy Generated (MWH)	327,278	1,548,540	13,163,603
19. Unit Service Factor	96.6%	65.1%	68.7%
20. Unit Availability Factor	96.6%	65.1%	68.7%
21. Unit Capacity Factor (Using MDC Net)	85.4%	51.4%	52.2%
22. Unit Capacity Factor (Using DER Net)	81.8%	49.2%	50.0%
23. Unit Forced Outage Rate	3.4%	7.9%	19.7%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling, March 7, 1981, 9 weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

* Turbine Rating: 565.7 MWe
Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe

8009190 504

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-0331
UNIT Duane Arnold Energy
DATE September 15, 1980
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MONTH August, 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	482	17	267
2	477	18	465
3	468	19	462
4	488	20	462
5	491	21	481
6	483	22	492
7	471	23	380
8	467	24	157
9	472	25	394
10	475	26	467
11	487	27	475
12	493	28	484
13	490	29	473
14	492	30	481
15	442	31	478
16	26		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH August, 1980

DOCKET NO. 050-0331
UNIT NAME Duane Arnold Energy Ctr.
DATE September 15, 1980
COMPLETED BY J. Van Sickel
TELEPHONE 319-851-5611

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14.	800816	F	19.9	A	1				Shutdown to repair nitrogen leak on feedwater check valve.
15.	800823	S	0	H	4				Power was reduced in order to perform a control rod sequence exchange.
16.	800824	F	5.1	B	4				Power was reduced and the generator taken off line in order to perform turbine overspeed trip testing.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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Docket No. 050-0331
Unit Duane Arnold Energy Center
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Telephone 319-851-5611

REFUELING INFORMATION

1. Name of facility.

   A. Duane Arnold Energy Center

2. Scheduled date for next refueling shutdown.

   A. March 7, 1981

3. Scheduled date for restart following refueling.

   A. May 3, 1981

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   A. No

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

   A. N/A

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   A. No licensing action is anticpated.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

   A. a) 368 b) 364

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   A. 2050

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   A. 1998

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MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
8-3-80	Reactor Building Radiation Monitoring	RM 4131B	Replaced power supply
8-4-80	Primary Containment H & V	RE 8102B	Replaced detector tube
8-5-80	Containment Atmospheric Control	AN 8181A	Installed new chemicals
8-6-80	Reactor Building Exhaust Radiation Monitoring	RM 7606A	Installed new integrated circuits
8-6-80	Containment Atmospheric Control	AN 8181B	Installed new chemicals
8-7-80	Primary Containment H & V	RE 8102A	Replaced detector tube
8-8-80	Containment Atmospheric Control	AN 8181B	Installed new chemicals
8-8-80	Containment Atmospheric Control	AN 8181A	Replaced power supply
8-13-80	RCIC	MOV 2405	Repositioned latching pin
8-13-80	Control Building H & V	Chiller 1V-CH-1A	Replaced oil pump
8-14-80	Containment Atmospheric Control	AN 8181A	Replaced power supply and solenoid valve
8-14-80	Primary Containment H & V	RE 8102B	Replaced detector tube

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MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
8-21-80	Containment Atmospheric Control	AN 8181A	Installed new chemicals
8-21-80	Control Building H & V	DO-6107A	Replaced operator connecting pin
8-21-80	River Water Supply	1P-117A	Rebuilt pump
8-29-80	Containment Atmospheric Control	AN 8181A	Installed new chemicals

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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

8-1 At the beginning of the report period the plant was operating at 523 MWe.

8-1 During normal operation drywell particulate radiation recorders RR 4379A & B were found downscale.

RO Report 80-038

8-9 A lightning strike on or near the site caused the "A" recirc pump to run back to 45%, the "B" well water pump to trip and all plant ARMs to alarm. Plant conditions were quickly restored to normal.

8-12 During normal operation the "A" river water supply system was taken out of service to interchange the motors on pumps 1P-117A and 1P-117C.

RO Report 80-040

8-13 During surveillance testing containment spray permissive (Drywell pressure not low) PS 4311C tripped at 2.3 PSIG. The required setpoint is 1.5 ± .5 PSIG.

RO Report 80-042

8-13 During surveillance testing RCIC turbine trip throttle valve MOV 2405 would not open following a RCIC isolation.

RO Report 80-043

8-14 Periodic venting of the containment became necessary due to a nitrogen leak inside the drywell.

8-15 A power reduction was begun at 1845 hours in preparation for a plant outage to repair the nitrogen leak inside the drywell.

8-16 The generator was taken off line at 0210 hours and the reactor was sub-critical at 0340 hours.

8-16 A nitrogen leak was found on the operator for the "B" feedwater check valve. The leak was repaired and preparations for plant startup were begun. The reactor was critical at 1835 hours. The generator was placed on the line at 2203 hours and a power increase begun.

8-20 During surveillance testing, "B" core spray system header to top of core plate high differential pressure switch PDIS 2139 tripped at 2.75 PSID. The required setpoint is 2.46 ± .25 PSID.

RO Report 80-039

80-20 During normal operation control building exhaust isolation damper 1V-AD-31A would not close.

RO Report 80-041

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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

8-21 The plant was operating at 520 MWe.

8-23 Power was reduced and a control rod sequence exchange completed.

8-23 A power decrease was begun at 2145 hours in preparation for plant outage to perform turbine overspeed trip testing.

8-24 The generator was taken off the line at 0332 hours. The generator was placed back on the line at 0357 hours and again removed from the line at 0415 hours. The generator was placed on the line at 0859 hours and a power increase begun.

8-24 During a power ascension the "B" rod block monitor indicated downscale and was bypassed as per Technical Specifications and control rod withdrawals were continued.

RO Report Pending

8-27 The plant was operating at 522 MWe.

8-28 During normal operation it was determined that two new fire protection surveillance requirements were not completed on time due to design change work not being completed.

RO Report Pending