

OPERATING DATA REPORT

DOCKET NO. 050-0331
 DATE 3-14-80
 COMPLETED BY J. Van Sickle
 TELEPHONE 319-851-5611

OPERATING STATUS

1. Unit Name: Duane Arnold Energy Center
2. Reporting Period: February, 1980
3. Licensed Thermal Power (MWt): 1658
- * 4. Nameplate Rating (Gross MWe): 565 (Turbine Rating)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	696	1,440	44,520
12. Number Of Hours Reactor Was Critical	200.1	944.1	31,508.2
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	200.1	944.1	30,763.2
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	243,768	565,944	12,181,008
17. Gross Electrical Energy Generated (MWH)	84,693	428,336	12,852,702
18. Net Electrical Energy Generated (MWH)	79,173	401,349	12,016,413
19. Unit Service Factor	28.7%	65.6%	69.1%
20. Unit Availability Factor	28.7%	65.6%	69.1%
21. Unit Capacity Factor (Using MDC Net)	22.1%	54.1%	52.4%
22. Unit Capacity Factor (Using DER Net)	21.1%	51.8%	50.2%
23. Unit Forced Outage Rate	0%	0%	20.4%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

Refueling, February 9, 1980, 10 weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup: April 10, 1980

* Turbine Rating: 565.7 MWe

Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe

(9/77)

8003180373

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-0331

UNIT Duane Arnold Energy
Center

DATE 3-14-80

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MONTH February, 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>418</u>
2	<u>412</u>
3	<u>410</u>
4	<u>408</u>
5	<u>408</u>
6	<u>407</u>
7	<u>405</u>
8	<u>390</u>
9	<u>34</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u></u>
31	<u></u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1980

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UNIT NAME Duane Arnold Energy Center

DATE 3-14-80

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No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1.	800209	S	495.9	C	3				Plant was shutdown for refueling on 2-9-80. While shutting down and at approximately 6% power, a reactor scram and turbine trip occurred during turbine control valve testing.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵
Exhibit I - Same Source

REFUELING INFORMATION

Docket No. 050-0331
Unit Duane Arnold Energy Center
Date 3-14-80
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1. Name of facility.
A. Duane Arnold Energy Center
2. Scheduled date for next refueling shutdown.
A. Spring, 1981
3. Scheduled date for restart following refueling.
A. Unknown.
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
A. No
5. Scheduled date(s) for submitting proposed licensing action and supporting information.
A. N/A
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
A. No licensing action is anticipated.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
A. a) 0 b) 732
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
A. 2050
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
A. 1998

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

- 2-1 At the beginning of the report period the plant was operating at 448 MWe. The plant was in base loaded coast down operation.
- 2-5 During surveillance testing RCIC barometric condenser condensate pump 1P-228 was found inoperable due to a defective contactor coil. The coil was replaced and the RCIC system tested with satisfactory results.

R0 Report 80-005

- 2-6 A fire watch was improperly removed from the cable spreading room while the room CO-2 system was out of service for maintenance.

R0 Report 80-002

- 2-8 A power reduction was begun at 2018 hours in preparation for a refueling outage.
- 2-9 While doing turbine control valve testing at approximately 6% power, a reactor scram occurred. Preparations for putting the plant in a cold shutdown were begun.
- 2-10 The reactor was in cold shutdown at 0400 hours. Torus draining began at 2021 hours. Drywell head detensioning was begun at 2100 hours.
- 2-11 The drywell head was removed at 0200 hours.
- 2-11 During surveillance testing PS-4555 (RHR low pressure permissive) was found to have its trip setpoint out of tolerance.

R0 Report 80-007

- 2-14 The insulation head was removed at 0019 hours. The reactor head was removed at 2155 hours.
- 2-14 During the performance of schedule leak rate testing, inboard MSIV's 4412, 4418 and 4420 were found to have seat leakage and outboard MSIV's 4413, 4416, 4419, and 4421 were found to have combined seat and packing leakage in excess of 11.5 SCFH limit specified in Technical Specification Section 4.7.A.2.c.3. In addition, because of the outboard MSIV packing leakage, the total allowable leakage for Type B and C leak rate tests specified in Technical Specifications Section 4.7.A.2.c.2 was exceeded.

R0 Report 80-004

- 2-15 The steam dryer was removed from the reactor.
- 2-16 The moisture separator was removed from the reactor.

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NARRATIVE SUMMARY OF OPERATING EXPERIENCE

2-16 During surveillance testing core spray sparger break detection PDIS 2139 tripped at an out of tolerance value.

R0 Report 80-008

2-18 Core unloading was begun at 1615 hours.

2-19 A reactor scram occurred due to breaker 1A-103 being tripped. Seventeen control rods inserted in empty cells due to cooling water flow. The rods were withdrawn and core unloading resumed.

2-22 Core unloading was completed at 0445 hours.

2-22 During an inspection of the HPCI turbine, two reversing chambers were found to be cracked.

Unique Report Pending

2-27 During an inspection of the RCIC turbine both the governor end and pump end bearings were found partially wiped.

Unique Report Pending

MAJOR SAFETY RELATED MAINTENANCE

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DATE	SYSTEM	COMPONENT	DESCRIPTION
2-1-80	Containment Atmospheric Control	AR-4381A	Recalibrated unit
2-5-80	RCIC	1P-228	Replaced contactor coil
2-13-80	Main Steam Isolations	TIS-4446	Cleaned meter movement
2-15-80	Reactor Water Cleanup	V-27-11	Installed new hinge pin gasket, installed new pressure gasket
2-18-80	Standby Diesel Generator	1G-31	Adjusted coolant temperature switch
2-19-80	Control Rod Drive-Elect.	Rod select panel	Replaced rod 22-19 select switch