OPERATING DATA REPORT

DOCKET NO. 050-0331
DATE 1-15-80
COMPLETED BY J. Van Sickel
TELEPHONE 319-851-5611

OPERATING STATUS

1. Unit Name: Duane Arnold Energy Center
2. Reporting Period: December, 1979
3. Licensed Thermal Power (MWt): 1658
* 4. Nameplate Rating (Gross MWe): 565 (Turbine rating)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	8760	43,080
12. Number Of Hours Reactor Was Critical	744	6,915.3	30,564.1
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	744	6,832.1	29,819.1
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,045,416	9,067,920	37,129,560
17. Gross Electrical Energy Generated (MWH)	366,935	3,086,937	12,424,366
18. Net Electrical Energy Generated (MWH)	345,093	2,898,764	11,615,063
19. Unit Service Factor	100%	78%	69.2%
20. Unit Availability Factor	100%	78%	69.2%
21. Unit Capacity Factor (Using MDC Net)	90.1%	64.3%	52.4%
22. Unit Capacity Factor (Using DER Net)	86.2%	61.5%	50.1%
23. Unit Forced Outage Rate	0%	22%	20.9%

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling, February 9, 1980, 12 weeks

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

* Turbine Rating: 565.7 MWe
Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MWe

8001230 081

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-0331
UNIT Duane Arnold Energy Center
DATE 1-15-80
COMPLETED BY J. Van Sickel
TELEPHONE 319-851-5611

MONTH December, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	344	17	473
2	434	18	470
3	490	19	467
4	473	20	467
5	468	21	463
6	481	22	465
7	482	23	460
8	481	24	460
9	471	25	460
10	452	26	462
11	501	27	462
12	481	28	461
13	477	29	460
14	475	30	451
15	471	31	458
16	460		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December, 1979

DOCKET NO. 050-0331
UNIT NAME Duane Arnold Energy Ctr.
DATE 1-15-80
COMPLETED BY J. Van Sickel
TELEPHONE 319-851-5611

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
22	791201	S	0	F	4	N/A	RB	Conrod	Power reduction to withdraw control rods

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

(9/77)

Docket No. 050-0331
Unit Duane Arnold Energy Center
Date 1-5-80
Completed by J. Van Sickel
Telephone 319-851-5611

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

12-1 At the beginning of the report period the plant was operating at 521 MWe. At 0043 hours a load reduction was begun in order to perform control rod withdrawals. Withdrawals were completed and a load increase was begun at 0604 hours.

12-2 During a routine instrument check, FR-4133, off-gas stack flow recorder, was found to be indicating downscale due to problems with its associated transmitter, DPT 4133.

ETSV Report79-3

12-3 The plant was operating at 524 MWe.

12-21 During normal operation, pen 1 (drywell pressure) on PR 4384 was found to be indicating downscale.

RO Report 79-038

12-25 During the performance of control room panel surveillance testing, a fuse was blown in the power supply to annunciator panels 1C-03, 04 and 05. This caused a loss of both channels of the Core Spray Sparger Break Detection Alarm System. Since one channel is required to be functionaly at all times, T.S.3.2.B was violated. Both audio and visual alarm indication was unavailable for approximately 3 hours. The need to bring the reactor to cold shutdown condition within 24 hours was recognized.

RO Report 79-037

12-31 1G-31 Standby diesel generator was declared inoperable due to low lube oil level.

RO Report Pending

12-31 At the end of the report period the plant was in coast down operation with a refueling outage scheduled to begin February 9, 1980.

Docket No. 050-0331
Unit Duane Arnold Energy Center
Date 1-15-80
Completed by J. Van Sickel
Telephone 319-851-5611

MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
12-7-79	Containment Atmospheric Control	AN-8181B	Installed new flowmeter
12-10-79	Neutron Monitoring-	APRM Channel "E"	Replaced zener diode
12-14-79	RHR	Pipe support GBB-5-H19	Constructed and installed support
12-20-79	Containment Atmospheric Control	AN-8181B	Rejuvenated cell
12-20-79	RCIC	MOV 2515	Packed valve
12-22-79	Containment Atmospheric Control	PR-4384	Replaced slide wire
12-26-79	Containment Atmospheric Control	RE 8101B	Replaced detector
12-28-79	Containment Atmospheric Control	SV-8106B	Replaced solenoid

Docket No. 050-0331
Unit Duane Arnold Energy Center
Date 1-15-80
Completed by J. Van Sickel
Telephone 319-851-5611

REFUELING INFORMATION

1. Name of facility.

   A. Duane Arnold Energy Center

2. Scheduled date for next refueling shutdown.

   A. February 9, 1980

3. Scheduled date for restart following refueling.

   A. May 3, 1980

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   A. Yes. MCPR and MAPLHGR operating limits as derived from transient and accident analyses.

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

   A. January 18, 1980

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

   A. The reload will consist of 88 8 x 8 2 water rod bundles.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

   A. a) 368 b) 276

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

   A. 2050

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

   A. 1998