OPERATING DATA REPORT

13-79	2	
Van	Sick	e
-851-	<u>-5</u> 611	
	3-79 Van 851-	<u>3-79</u> Van Sick 851-5611

OPERATING STATUS

During August J. Final	and Contour	Notes	
1. Unit Name:Uane_Arnoid_Ener	INDIES	·	
2. Reporting Period:		· ·	
3. Licensed Thermal Power (MWt):	1658		
4. Nameplate Rating (Gross MWe): <u>505</u>	(Turbine Rating)		
5. Design Electrical Rating (Net MWe):	538		
6. Maximum Dependable Capacity (Gross M	/We): <u>545</u>		
7. Maximum Dependable Capacity (Net MW	Ve):	L	·····
8. If Changes Occur in Capacity Ratings (Ite	ems Number 3 Through 7) Sin	ce Last Report, Give R	easons:
			······
9. Power Level To Which Restricted If Any	v (Net MWe)		
0. Reasons For Restrictions. If Any:	(itel Mitte).		· · · · · · · · · · · · · · · · · · ·
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	179-1- 3.4	Y D	•.
· · ·	Inis Month	Yrto-Date	Cumulative
I. Hours In Reporting Period	672	1416	35,736
2. Number Of Hours Reactor Was Critical	0	7.5	23,620.3
3. Reactor Reserve Shutdown Hours	0	0	0
4. Hours Generator On-Line	0	<u> </u>	22,987
5. Unit Reserve Shutdown Hours	0	0	0
6. Gross Thermal Energy Generated (MWH)	0	0	28,061,640
7. Gross Electrical Energy Generated (MWH	I) <u> </u>	0	9,337,429
8. Net Electrical Energy Generated (MWH)	0	0	8,716,300
9. Unit Service Factor	0%	0%	64.3%
0. Unit Availability Factor	0%	0%	64.3%
Unit Canacity Factor (Using MDC Nat)	. 0%	0%	47.4%
1. One capacity Factor (Using MDC Net)			
2. Unit Capacity Factor (Using MDC Net)	0%	0%	45.3%

7903200380

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	050-0331		
UNIT	Duane Arnold Energy		
DATE	<u>3-13-79</u>		
COMPLETED BY	J. Van Sickel		
TELEPHONE	<u>319-851-5611</u>		

MONT	H February, 1979		
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6		22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	25	0
11	0	20	0
12	0		 0
13	0	20	<u>_</u>
14	0	29	
15	· 0 ·	30	
1.5		31	

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1979

050-0331 DOCKET NO. UNIT NAME Duane Arnold Energy DATE 3-12-79 COMPLETED BY J. Van Sickel TELEPHONE 319-851-5611

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Cude ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	780617	F	672	A	3	78-030 79-001	СВ	PIPEXX	Continuation of previous shutdown in which the recirculation system inlet nozzle safe ends were replaced. Start up was delayed while a flow restriction was removed from the N2B riser (See LER 79-001)
I F: For S: Sch (9/77)	rced eduled	Reaso A-Equ B-Mai C-Ref D-Reg E-Ope F-Adu G-Ope H-Out	n: uipment Fai intenance of ueling gulatory Re: erator Train ministrative erational Er her (Explain	lure (Ex f Test striction ing & Li ror (Exp)	(plain) cense Exa plain)	3 mination	Method I-Manu 2-Manu 3-Auto 4-Othe	l: ial scram. matic Scrain. r (Explain)	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I - Same Source

De ke	t No.		050	-03	331		
Unit I	Duar	ne ¯	Arno	ld	Er	nergy	Center
Date ⁻	Marc	:h	13,	197	'9		
Comple	eted	by	J.	Va	in	Sicke	2]
Telep	hone		319	-85	51-	-5611	

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

- 2-1 At the beginning of the report period the reactor was in the cold shutdown condition with the reactor open to investigate and remove a blockage discovered in the N2B riser which was making jet pumps #3 and #4 inoperable.
- 2-3 A fiberscope inspection of the N2B riser confirmed that a lead shielding plug had been left in the pipe during safe end replacement program and this plug was causing the flow blockage in the #3 and #4 jet pumps.
- 2-9 Preparations were in progress to remove jet pumps #3 and #4 from the reactor to allow for removal of the lead shielding plug from the N2B riser pipe.
- 2-11 Jet pumps #3 and #4 were removed from the reactor and placed in the spent fuel pool.
- 2-12 Six sections of the 10 part lead shielding plug were removed from the N2B riser pipe. The perimeter can which had been around the plug was also removed.
- 2-13 Two plug sections were removed from jet pump #3 and 1 plug section was removed from jet pump #4. In addition, the perimeter can backing plate was removed from jet pump #4.
- 2-15 The service platform was removed from the vessel flange and an invessel fuel storage rack installed.
- 2-16 Shoot out steel was removed from under the reactor vessel in preparation for removing control rod drives. Selected CRD's and fuel cells were removed while searching for the tenth lead shield plug section.
- 2-21 The tenth lead shielding plug section was found and removed from the reactor vessel.
- 2-23 All fuel was in position in the core and vessel reassembly was begun. The invessel fuel storage rack was removed.
- 2-24 Control rod friction testing was completed.
- 2-26 The moisture separator and steam dryer were installed in the reactor vessel. A subsequent review of core verification videotapes revealed one fuel cell was not properly seated. The steam dryer and moisture separator were again removed from the vessel.
- 2-27 Fuel support piece 06-15 was properly positioned and reactor reassembly was begun.
- 2-28 The moisture separator and steam dryer were installed in the reactor vessel. The reactor vessel head was set in place and vessel heat up for head stud tensioning was begun with RHR pump heat.

Docket No. <u>050=0331</u> Unit <u>Duane Arnold Energy Cen</u>te Date <u>March 13, 1979</u> Completed by <u>J. Van Sickel</u> Telephone <u>319-851-5611</u>

MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
2-5-79	Reactor vessel recirculation	FT 4631D and FT 4632 D	Recalibrated flow transmitters
2-7-79	Mái n steam isolations and ADS	TIS 4477, 4478, 4479 and 4480	Installed new capacitors in power supplies
2-8-79	Reactor building H & V	SV 7605B	Repaired flange leak
2-14-79	Sta n dby diesel generators	1G-21 stàndby diesel	Replaced exhaust manifold gaskets
2-16-79	RHR cooling water	PSV 1982	Installed new relief valve
2-19-79	Fuel handling	Refuel bridge monorail hoi <u>st</u>	Installed new cable
2-20-79	Fuel handling	Refuel bridge	Repaired power leads to main grapple
2-22-79	Containment atmosphere	SV4332D, SV4413B, SV4333B and SV4334B	Replaced reed switches
2-22-79	CRD hydraulic	CRD serial numbers 4777, 3996 and 3975	CRDs were removed from the reactor, rebuilt and placed in storage
2-23-79	CRD hydraulic	Control rod 22-23	A rebuilt CRD was installed at this location
2-25-79	CRD hydraulic	Control rod 30-23	A rebuilt CRD was installed at this location
2-25-79	CRD hydraulic	CRD 10-31 and CRD 02-23	Replaced position indicating probe tube
2-25-79	Reactor protection	LIS 4535 and LIS 4537	Disassembled, cleaned, reassembled and calibrated.
2-25-79	CRD hydraulic	CRD serial numbers 4593 and 4531	CRDs were removed from the reactor, rebuilt and placed in storage
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MAJOR SAFETY RELATED MAINTENANCE

Docket No.	050-0331	æ
Unit Duane	Arnold Energy	Cente
Date March	13, 1979	×
Completed	p y <u>J. Van Sicke</u>	<u>l.</u>
Telephone <u>(</u>	319-851-5611	

DATE	SYSTEM	COMPONENT	DESCRIPTION
2-25-79	Nuclear Boiler	Jet pump inlet mixer assembly for jet pumps #3 and #4	Assembly was removed and placed in the spent fuel pool for inspection
2-28-79	CRD hy draulic	Control Rod 34-31	A rebuilt CRD was installed at this location
2-28-79	CRD hydraulic	Control Rod 38-23	A rebuilt CRD was installed at this location

REFUELING INFORMATION

Unit Duane Arnold Energy Center Date March 13, 1979 Completed by J. Van Sickel Telephone 319-851-5611

050-0331

Docket No.

1. Name of facility.

A. Duane Arnold Energy Center

- 2. Scheduled date for next refueling shutdown.
 - A. Unknown. Under review due to present extended outage.
- 3. Scheduled date for restart following refueling.
 - A. Unknown. Under review due to the present extend outage.
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
 - A. MCPR and MAPLHGR operating limits as derived from transient and accident analyses.
- 5. Scheduled date(s) for submitting proposed licensing action and supporting information.

A. Unknown.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

A. The reload will consist of up to $100 \ 8 \ x \ 8 \ 2$ water rod bundles.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

A. a) 368 b) 276

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

A. 1998

A. 2050