
1.6 Material Referenced

Table 1.6-1—Reports Referenced, contains a list of AREVA NP reports submitted to the NRC that are incorporated by reference in the U.S. EPR FSAR. A COL applicant that references the US EPR design certification will include any site-specific topical reports that are incorporated by reference as part of the COL application in Table 1.6-1.

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
ANF-89-060P-A ANF-89-060NP-A Supplement 1	Generic Mechanical Design Report High Thermal Performance Spacer and Intermediate Flow Mixer	3/28/91	4.2
ANP-10263P-A ANP-10263NP-A	Codes and Methods Applicability Report for the U.S. EPR	11/06/07	4, 5.1, 15, 16, and 19
ANP-10264NP-A	U.S. EPR Piping Analysis and Pipe Support Design Topical Report	11/07/08	3.6, 3.7, 3.8, 3.9, 3.10, 3.12, App. 3A, and App. 3C
ANP-10264NP Revision 1	U.S. EPR Piping Analysis and Pipe Support Design Topical Report	5/10	3.9 and 3.12
ANP-10266A Revision 1	AREVA NP Inc. Quality Assurance Plan (QAP) for Design Certification of the U.S. EPR Topical Report	06/18/07	7.1, 9.5, 14.3, 15.0, 17.0, 17.4, 17.5, 18.1, 18.7, and 18.11
ANP-10268P-A ANP-10268NP-A	U.S. EPR Severe Accident Evaluation Topical Report	2/26/08	6.2.5, 15.4, 19.1, and 19.2
ANP-10269P-A ANP-10269NP-A	The ACH-2 CHF Correlation for the U.S. EPR Topical Report	3/10/08	4.4, 5, 7, 15, and 19
ANP-10272 Revision 3	Software Program Manual TELEPERM XS™ Safety Systems Topical Report	10/10	7.1 and 7.6
ANP-10275P-A ANP-10275NP-A	U.S. EPR Instrument Setpoint Methodology Topical Report	2/26/08	7.1, 7.2, 7.3 and 16
ANP-10278P ANP-10278NP Revision 1	U.S. EPR Realistic Large Break Loss of Coolant Accident Topical Report	01/10	6.2 and 15
ANP-10282P ANP-10282NP	POWERTRAX/E Online Core Monitoring Software for the U.S. EPR Technical Report	11/27/07	4.4
ANP-10283P Revision 1 ANP-10283NP Revision 1	U.S. EPR Pressure-Temperature Limits Methodology for RCS Heat-Up and Cool-Down Technical Report	4/30/09	5.3 and 16
ANP-10285P ANP-10285NP	U.S. EPR Fuel Assembly Mechanical Design Topical Report	10/02/07	4
ANP-10286P ANP-10286NP	U.S. EPR Rod Ejection Accident Methodology Topical Report	11/20/07	4.3 and 15

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
ANP-10287P ANP-10287NP	Incore Trip Setpoint and Transient Methodology for U.S. EPR Topical Report	11/27/07	4.3, 4.4, 7.1, 7.2, 15.0, 15.1, 15.2, 15.3, 15.4, 15.6, 16
ANP-10288P ANP-10288NP Revision 1	U.S. EPR Post-LOCA Boron Precipitation and Boron Dilution Technical Report	01/10	15
ANP-10290 Revision 1	AREVA NP Environmental Report Standard Design Certification	9/11/09	19.2
ANP-10291P ANP-10291NP	Small Break LOCA and Non-LOCA Sensitivity Studies and Methodology Technical Report	5/09	15
ANP-10292 Revision 1	U.S. EPR Conformance with Standard Review Plan (NUREG-0800) Technical Report	5/09	1.9
ANP-10293, Revision 3	U.S. EPR Design Features to Address GSI-191 Technical Report	3/11	6.3 and 15.6.5.4.3
ANP-10294 Revision 1	U.S. EPR Reactor Coolant Pump Motor Flywheel Structural Analysis Technical Report	3/09	5.4.1.6.6
ANP-10295 Revision 1	U.S. EPR Security Design Features	10/09	13.6
ANP-10296	U.S. EPR Design Features that Enhance Security	12/08	13.6
ANP-10299P Revision 2	Applicability of AREVA NP Containment Response Evaluation Methodology to the U.S. EPR for Large Break LOCA Analysis	12/09	6.2.1 and 6.2.2
ANP-10304 Revision 4	U.S. EPR Diversity and Defense in Depth Assessment Technical Report	6/11	1.9, 7.1, 7.2, 7.3, 7.8, 18.7, 19.1
ANP-10306P	Comprehensive Vibration Assessment Program for U.S. EPR Reactor Internals Technical Report	12/09	3.9.2.1.1, 3.9.2.3, 3.9.2.4, and 3.9.2.7
ANP-10318P	Pipe Rupture External Loading Effects on U.S. EPR Essential Structures, Systems, and Components Technical Report	3/11	3.6.2

**Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
ANP-10309P ANP-10309NP Revision 3	U.S. EPR Protection System Technical Report	06/11	4.6, 7.1, 7.2, and 7.3
ANP-10310P Revision 1	Methodology for 100% Combinatorial Testing of the U.S. EPR™ Priority Module Technical Report	03/11	7.1
ANP-10315P Revision 1	U.S. EPR Protection System Surveillance Testing and Teleperm XS Self-Monitoring Technical Report	6/11	7.1,7.3
ANP-10317	Design Requirements for the U.S. EPR Aircraft Hazard Protection Structures	5/11	19.2.7.4
ANP-10318P	Pipe Rupture External Loading Effects on U.S. EPR Essential Structures, Systems, and Components Technical Report	3/11	3.6.2
BAW-10132-A	Analytical Methods Description – Reactor Coolant System Hydrodynamic Loadings During a Loss-of-Coolant Accident	7/20/79	App. 3C
BAW-10133P-A BAW-10133-A Revision 1, Addendum 1 and 2	Mark-C Fuel Assembly LOCA-Seismic Analysis	10/30/00	4.2
BAW-10147P-A BAW-10147-A Revision 1	Fuel Rod Bowing in Babcock & Wilcox Fuel Designs	6/28/83	4.2, 4.4
BAW-10156-A Revision 1	LYNXT, Core Transient Thermal-Hydraulic Program	8/18/93	4
BAW-10163P-A BAW-10163-A	Core Operating Limit Methodology for Westinghouse Designed PWRs	6/2/89	4.3 and 16
BAW-10164P-A BAW-10164NP-A Revision 6	RELAP5/MOD2-B&W - An Advanced Computer Program for Light Water Reactor LOCA and Non-LOCA Transient Analysis	11/20/07	3.9.1, 6.2, and 8.4
BAW-10168P-A BAW-10168-A Revision 3	BWNT Loss-of-Coolant Accident Model for Recirculating Steam Generator Plants	1/31/97	6.2

Table 1.6-1—Reports Referenced
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Report No. (See Notes 1, 2, and 3)	Title	Date Submitted to NRC	FSAR Section Number(s)
BAW-10169P-A BAW-10169-A	B&W Safety Analysis Methodology for Recirculating Steam Generator Plants	11/28/89	6.2
BAW-10172P-A BAW-10172NP-A	Mark-BW Mechanical Design Report	12/19/89 (Note 4)	4.2
BAW-10183P-A BAW-10183-A	Fuel Rod Gas Pressure Criterion (FRGPC)	7/24/95	4.2 and 4.4
BAW-10186P-A BAW-10186NP-A Revision 2	Extended Burnup Evaluation	1/30/04	4.2 and 4.4
BAW-10199P-A BAW-10199-A	The BWU Critical Heat Flux Correlations	8/19/96	4.4
BAW-10221P-A BAW-10221-A	NEMO-K A Kinetics Solution in NEMO	9/17/88	4.3 and 16
BAW-10227P-A BAW-10227NP-A Revision 1	Evaluation of Advanced Cladding and Structural Material (M5) in PWR Reactor Fuel	6/18/03 (Note 4)	4.1, 4.2, and 15
BAW-10231P-A BAW-10231NP-A Revision 1	COPERNIC Fuel Rod Design Computer Code	9/30/04	4 and 16
BAW-10239P-A BAW-10239NP-A	Advanced Mark-BW Fuel Assembly Mechanical Design Topical Report	10/5/04	4.2
BAW-10240P-A BAW-10240NP-A	Incorporation of M5™ Properties in Framatome ANP Approved Methods	8/17/04	4.2
BAW-10243P-A BAW-10243NP-A	Statistical Fuel Assembly Hold Down Methodology	12/21/05	4.4
BAW-10252P-A BAW-10252NP-A	Analysis of Containment Response to Postulated Pipe Ruptures using GOTHIC	12/21/05	6.2
BAW-2241P-A BAW-2241NP-A Revision 2	Fluence and Uncertainty Methodologies	11/20/07	4.1 and 5.3
EMF-2110(NP)(A) Revision 1	TELEPERM XS: A Digital Reactor Protection System	7/12/00	7.1, 7.2, 19.1
EMF-96-029(P)(A) EMF-96-029(NP)(A)	Reactor Analysis Systems for PWRs, Volume 1 -Methodology Description, Volume 2 -Benchmarking Results	1/13/97	4.3 and 16
TN-Rack 0101(P)	U.S. EPR New and Spent Fuel Storage Rack Technical Report	12/09	9.1.1, and 9.1.2

Notes:

1. P – Denotes document is proprietary.
2. NP – Denotes the document is non-proprietary. Any reports that do not have a “P” or “NP” after the numerical designation are also non-proprietary.
3. “-A” denotes the published version of the report following issuance of the NRC Safety Evaluation Report approving the topical report.
4. This is the date that the NRC approved this topical report. The letter transmitting the approved version to NRC is not publicly available.
5. A non-proprietary version of this report is not available.
6. This report was submitted to support the NRC’s review of the TELEPERMTM XS system and was found acceptable as part of the NRC SER for EMF-2110(NP) Revision 1 (Reference NRC letter dated May 5, 2000, accession number ML003711856). An approved version of this report was not developed.