



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 22, 2011

Mr. Paul A. Harden
Site Vice President
FirstEnergy Nuclear Operating Company
Beaver Valley Power Station
Mail Stop A-BV-SEB1
P.O. Box 4, Route 168
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2 - REQUEST FOR
ADDITIONAL INFORMATION REGARDING RELIEF REQUESTS VRR3 AND
VRR4 (TAC NOS. ME5749, ME5750, AND ME5752)

Dear Mr. Harden:

By letter dated February 21, 2011, FirstEnergy Nuclear Operating Company (the licensee) submitted relief request VRR3 for authorization of proposed alternatives to position verification testing of certain solenoid-operated valve assemblies performed in accordance with Paragraph ISTC-3700 of the American Society of Mechanical Engineers Operation and Maintenance Code (ASME OM Code) for the Beaver Valley Power Station, Unit No. 1 (BVPS-1) fourth inservice testing program (IST) interval and for the BVPS-2 third IST program interval. By another letter also dated February 21, 2011, the licensee also submitted relief request VRR4 for authorization of a proposed alternative to testing of pressurizer safety valve assemblies performed in accordance with Appendix I, Paragraph 1-1320(a) of the ASME OM Code for the BVPS-2 third IST program interval.

The Nuclear Regulatory Commission (NRC) staff is reviewing the submittal and has determined that additional information is needed to complete its review. The specific questions are found in the enclosed request for additional information (RAI). The NRC staff is requesting a response to the RAI by September 16, 2011.

If you have any questions regarding this issue, please contact me at (301) 415-1016.

Sincerely,

A handwritten signature in black ink, appearing to read "Nadiyah S. Morgan for".

Nadiyah S. Morgan, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosure:

RAI

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION
REGARDING RELIEF REQUESTS VRR3 AND VRR4
INSERVICE TESTING PROGRAM
FIRSTENERGY NUCLEAR OPERATING COMPANY
BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-334 AND 50-412

By letter dated February 21, 2011(Agencywide Documents Access and Management System (ADAMS) Accession No. ML110550162), FirstEnergy Nuclear Operating Company (the licensee) submitted relief request VRR3 for authorization of proposed alternatives to position verification testing of certain solenoid-operated valve assemblies performed in accordance with Paragraph ISTC-3700 of the American Society of Mechanical Engineers Operation and Maintenance Code (ASME OM Code) for the Beaver Valley Power Station, Unit No. 1 (BVPS-1) fourth inservice testing program (IST) interval and for the BVPS-2 third IST program interval. By another letter also dated February 21, 2011 (ADAMS Accession No. ML110560014), the licensee also submitted relief request VRR4 for authorization of a proposed alternative to testing of pressurizer safety valve assemblies performed in accordance with Appendix I, Paragraph 1-1320(a) of the ASME OM Code for the BVPS-2 third IST program interval.

To complete its review, the U.S. Nuclear Regulatory Commission staff requests the following additional information:

Request VRR3:

1. For the valves subject to exercise testing, describe:
 - i. the exercise test method;
 - ii. the parameters that are measured or observed; and
 - iii. the criteria for evaluating the results.
2. Have any evaluations been performed, such as probabilistic assessment risk and expert panel review, to assess the change in aggregate risk associated with relaxing the interval for verification of obturator movement to as long as 60 months? If so, what were the results?

Request VRR4:

1. In Section 5.0, under Basis for Use, the licensee stated that the as-found set-pressure acceptance criteria for the BVPS-2 pressurizer safety valves is +/- 3% of nameplate set pressure. However, from reviewing Technical Specification 3.4.10.1, it appears that the acceptance criteria is more restrictive (i.e., 2467.6 +/- 2.3% psig). Explain this discrepancy. Does this affect the analysis of past operating history for the valves that the licensee presented as its "Basis for Use" to support the future use of Code Case OMN-17?

2. Will the licensee continue testing the removed valve prior to starting up after every refueling outage?
3. In the third paragraph, first sentence, under Basis for Use, the licensee refers to Code Case OMN-17 paragraphs (e) and (f). This appears to be a typographical error. Did you mean paragraphs (d) and (e)?

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/ra/ (BVaidya for)
Nadiyah S. Morgan, Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosure:

RAI

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ADAMS Accession Number: ML112310737

*See email dated August 11, 2011

OFFICE	LPL1-1/PM	LPL1-1/LA	CPTB/BC	LPL1-1/BC	LPL1-1/PM
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