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 FACIL: 50-331 Duane Arnold Energy Center, Iowa Electric Light & Pow 05000331  
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 MCGAUGHY, R.W. Iowa Electric Light & Power Co.  
 RECIP. NAME: RECIPIENT AFFILIATION  
 DENTON, H. Office of Nuclear Reactor Regulation, Director

SUBJECT: Forwards scope of plant mods & projected completion dates  
 needed to implement 840316 request for Tech Spec changes to  
 incorporate findings of NRC SER.

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Iowa Electric Light and Power Company

August 24, 1984  
NG-84-3601

Mr. Harold Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Subject: Duane Arnold Energy Center  
Docket No: 50-331  
Op. License No: DPR-49  
Schedule for 10 CFR Part 50, Appendix J  
Modifications

References: 1) Letter, R. W. McGaughy (IE) to H. R. Denton  
(NRC), March 16, 1984 (NG-84-0668), RTS-112B  
2) Safety Evaluation Report, D. Eisenhut (NRC)  
to L. Liu (IE), January 17, 1984

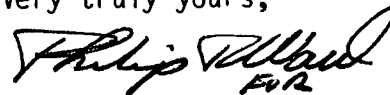
Dear Mr. Denton:

By our letter dated March 16, 1984 (Reference 1), Iowa Electric Light and Power Company requested revision to the Technical Specifications (TS) for the Duane Arnold Energy Center (DAEC). The TS revisions were required to incorporate the findings of the NRC staff's Safety Evaluation Report (SER) (Reference 2).

During the review of Reference 2, we determined certain plant modifications were needed in order to implement the TS changes required by the NRC staff. We discussed the scope of the modifications with our NRC Project Manager in a March 15 telephone call and agreed to provide, by September 15, 1984, a schedule for their completion. We also agreed that our Integrated Plan does not include these modifications; therefore, the Integrated Plan required revision to include the additional work.

The enclosure to this letter discusses the scope of the needed modifications and the projected completion dates. Should you have any questions, please feel free to call me.

Very truly yours,



Richard W. McGaughy  
Manager, Nuclear Division

RWM/MSG/dmb\*

Enclosure

cc: M. Grim  
S. Swails  
L. Liu  
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Iowa Electric Light and Power Company  
Duane Arnold Energy Center  
Docket No.: 50-331  
Operating License No.: DPR-49

Background

On January 17, 1984, the NRC staff issued a Safety Evaluation Report (SER) which dispositioned relief requests tendered by our letters of October 13, 1975, August 29, 1978, and November 5, 1981. The SER requested that Iowa Electric submit, within 60 days, Technical Specification (TS) changes which reflected the Staff's evaluation of our relief requests. On March 16, 1984, Iowa Electric submitted proposed Technical Specification (TS) changes (RTS-112B) for the Duane Arnold Energy Center (DAEC).

We noted in our March 16 letter that certain plant modifications were required as a result of the SER findings. We had discussed the scope of the modifications with our NRC Project Manager and agreed to provide, by September 15, 1984, a schedule for their completion. Addressed below are the areas which require modification and the schedule for completion.

Proposed Modifications

1. Penetration No.: N-213A, B

Type or System: Expansion Bellows

Area Requiring Modification: Torus Drain Lines

Discussion: RTS-112B proposes that two expansion bellows in each of two (2) torus drain lines be leak tested. The torus drain lines are not designed in a manner to enable testing of the expansion bellows. The bellows require modification to allow for testing.

Schedule: The modification to the bellows will be completed by the Cycle 8/9 refueling outage.

2. Penetration No.: X-8

Type or System: Main Steam Line Drain

Area Requiring Modification: Valve M0-4423

Discussion: RTS-112B proposes that M0-4423, the inboard isolation valve for the main steam isolation valve drain line, be leak tested from the reactor vessel direction. To test from the reactor vessel direction, a flange and test connection must be installed to facilitate testing.

Schedule: Modifications to install a flange and test connection will be completed by the Cycle 8/9 refueling outage.

3. Penetration No.: X-10 and X-11

Type or System: RCIC and HPCI Steam Supply

Area Requiring Modification: Valves CV-2410 and CV-2211

Discussion: RTS-112B proposes that CV-2410 and CV-2211, the RCIC and HPCI inboard drain valves, be leak tested from the reactor vessel side. To leak test from the reactor vessel side, test connections for the valves must be relocated.

Schedule: Modifications to relocate the test connections will be completed by the Cycle 8/9 refueling outage.