

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8406140202 DOC. DATE: 84/06/11 NOTARIZED: NO DOCKET #
 FACIL: 50-331 Duane Arnold Energy Center, Iowa Electric Light & Pow 05000331
 AUTH. NAME AUTHOR AFFILIATION
 MCGAUGHY, R.W. Iowa Electric Light & Power Co.
 RECIP. NAME RECIPIENT AFFILIATION
 DENTON, H. Office of Nuclear Reactor Regulation, Director

SUBJECT: Responds to 840403 request for info re NUREG-0737, Item II.K.3.16, "Reduction of Changes of Failures of Safety/Relief Valves." Mods listed safety/relief valve simmer margin will be increased during next refueling outage.

DISTRIBUTION CODE: A046S COPIES RECEIVED: LTR 1 ENCL 0 SIZE: 2
 TITLE: OR Submittal: TMI Action Plan Rgmt NUREG-0737 & NUREG-0660

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTR	ENCL	RECIPIENT ID CODE/NAME	COPIES LTR	ENCL
	NRR ORB2 BC 01	7				
INTERNAL:	ELD/HDS2	1	0	IE/DEPER DIR 33	1	
	IE/DEPER/EPB	3	3	IE/DEPER/IRB	1	
	NRR ERICKSON, P	1	1	NRR PAULSON, W.	1	
	NRR/DHFS DEPY29	1	1	NRR/DL DIR 14	1	
	NRR/DL/ORAB 18	3	3	NRR/DSI/ADRS 27	1	
	NRR/DSI/AEB	1	1	NRR/DSI/ASB	1	
	NRR/DSI/RAB	1	1	NRR/DST DIR 30	1	
	REG FILE 04	1	1	RGN3	1	
EXTERNAL:	ACRS 34	10	10	LPDR 03	1	
	NRC PDR 02	1	1	NSIC 05	1	
	NTIS	1				

TOTAL NUMBER OF COPIES REQUIRED: LTR 41 ENCL 0

Iowa Electric Light and Power Company

June 11, 1984

NG-84-2255

Mr. Harold Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Duane Arnold Energy Center
Docket No: 50-331
Op. License No: DPR-49
NUREG-0737 Item II.K.3.16
Reduction of Changes and Failures of SRVs

- References: 1. Letter, D. Vassallo to L. Liu, dated
April 3, 1984
2. Letter, BWROG-8134, dated March 31, 1984

Dear Mr. Denton:

The purpose of this letter is to provide information requested by Mr. Vassallo (Reference 1) concerning NUREG-0737, Item II.K.3.16. His letter requested we describe the modifications which have been performed and those we plan to perform to reduce Safety/Relief Valve (SRV) challenges and failures.

The following modifications which the NRC has found acceptable (Reference 1) have been made at the DAEC:

- (1) Low-Low Set Relief Logic System,
- (2) Lower Reactor Pressure Vessel (RPV) water level isolation setpoint for Main Steam Isolation Valve (MSIV) closure from low-low to low-low-low.

We plan to implement the following staff recommended modification which the NRC has found acceptable (Reference 1) during the next refueling outage:

- (3) Increase Safety/Relief Valve (SRV) simmer margin.

The analysis to increase the SRV simmer margin is being done as part of our Power Uprate Program. Therefore, installation of this modification will be implemented following NRC approval of our Power Uprate Program and the associated changes to the DAEC Technical Specifications prior to restart from the Cycle 7/8 refueling outage.

8406140202 840611
PDR ADOCK 05000331
PDR

Roak
1/0

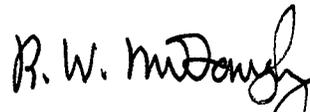
Mr. Harold Denton
June 11, 1984
NG-84-2255
Page Two

Additionally, with regard to item (4) of Reference 1, Iowa Electric has adopted a method which will reduce valve leakage which is not discussed in the Owners' Group report. This method addresses the procedures and leakage criteria utilized during the scheduled testing and maintenance. By employing more stringent leakage acceptance criteria during the scheduled testing and maintenance, the initial, as-installed valve leakage will be lower. It is then reasonable to assume that, with all other things being equal, the likelihood that the valve will develop threshold leakage (leakage which results in spurious opening) prior to the next testing cycle is reduced. This second method (more stringent leakage acceptance criteria during scheduled testing and maintenance) is employed at the DAEC.

While not identical to the method described in the Owners Group report, the DAEC's more stringent surveillance testing/maintenance procedures are comparable.

In addition to the staff recommended modifications listed above, we have lowered the reactor pressure isolation setpoint (from 880 psig to 850 psig) as described in the BWROG report (Reference 2). Since the remaining modifications described in the BWROG report do not significantly contribute to reducing SRV challenges and failures relative to their complexity and cost, we do not currently plan to implement them.

Very truly yours,



Richard W. McGaughy
Manager, Nuclear Division

RWM/SLS/dmb*

cc: S. Swails
L. Liu
S. Tuthill
M. Thadani
NRC Resident Office
Commitment Control No. 84-0086