

PROPOSED CHANGE RTS-235 TO THE DUANE ARNOLD ENERGY CENTER
TECHNICAL SPECIFICATIONS

The holders of license DPR-49 for the Duane Arnold Energy Center propose to amend Appendix A (Technical Specifications) to said license by deleting a current page and replacing it with the attached new page.

The proposed revision to Section 1.1.A reflects the change of the two recirculation loop MCPR from 1.04 to 1.07 and the single loop MCPR from 1.07 to 1.10.

List of Affected Pages

1.1-1

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SAFETY LIMIT	LIMITING SAFETY SYSTEM SETTING
1.1 FUEL CLADDING INTEGRITY	2.1 FUEL CLADDING INTEGRITY
<u>Applicability:</u>	<u>Applicability:</u>
Applies to the inter-related variables associated with fuel thermal behavior.	Applies to trip settings of the instruments and devices which are provided to prevent the reactor system safety limits from being exceeded.
<u>Objective:</u>	<u>Objective:</u>
To establish limits which ensure the integrity of the fuel cladding.	To define the level of the process variables at which automatic protective action is initiated to prevent the fuel cladding integrity safety limits from being exceeded.
<u>Specifications:</u>	<u>Specifications:</u>
A. <u>Reactor Pressure > 785 psig and Core Flow > 10% of Rated</u>	A. <u>Neutron Flux Trips</u>
The existence of a minimum critical power ratio (MCPR) less than 1.07 for two recirculation loop operation [1.10 for SINGLE LOOP OPERATION (SLO)] shall constitute violation of the fuel cladding integrity safety limit.	1. APRM High Flux Scram When In Run Mode
B. <u>Core Thermal Power Limit (Reactor Pressure ≤ 785 psig or Core Flow ≤ 10% of Rated)</u>	The APRM scram trip setpoint shall be as shown on Figure 2.1-1 and shall be:
When the reactor pressure is ≤ 785 psig or core flow is less than or equal to 10% of rated, the core thermal power shall not exceed 25 percent of rated thermal power.	$S \leq (0.58W + 62)$
	with a maximum setpoint of 120% rated power at 100% rated recirculation flow or greater for two recirculation loop operation and
	$S \leq (0.58W + 58.5)$
	for SLO.