LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT
	3) Type C Tests Type C tests shall be performed during each reactor shutdown for major refueling or other convenient interval but in no case at intervals greater than two years.
	4) Additional Periodic Tests Additional purge system isolation valve leakage integrity testing shall be performed at least once every six months in order to detect excessive leakage of the purge isolation valve resilient seats. The purge system isolation valves will be tested in three groups, by penetration: drywell purge exhaust group (CV-4302 and CV-4303), torus purge exhaust group (CV-4300 and CV-4301), and drywell/torus purge supply group (CV-4307, CV-4308 and CV-4306).
	e. Seal Replacement The T-ring inflatable seals for purge isolation valves CV-4300, CV-4301, CV-4302, CV-4303, CV-4306, CV-4307 and CV-4308 shall be replaced at intervals not to exceed four years. The baseline for this requirment shall be established during the 1982 refueling outage.
	f. Containment Modification Any major modification, replacement of a component which is part of the primary reactor containment boundary, or resealing a seal-welded door, performed after the preoperational leakage rate test shall be followed by either a Type A, Type B, or Type C test, as applicable, for the area affected by the modification.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT
	The measured leakage from this test shall be included in the test report. The acceptance criteria as appropriate, shall be met. Minor modifications, replacements, or resealing of seal welded doors, performed directly prior to the conduct of a scheduled Type A test do not require a separate test.
	g. Reporting
	The preoperational and periodic tests shall be the subject of a summary technical report submitted to the Commission approximately three months after the conduct of each test. The report will be titled "Reactor Containment Integrated Leakage Rate Test."
	The results of the periodic testing performed to satisfy the requirements of 4.7.A.2.d.4 shall be reported with the summary technical report prepared to provide the results of the testing performed in accordance with Section 4.7.A.2.d.3.

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT
must be removed from the run mode within 24 hours.	functionally tested once per operating cycle in conjunction with specification 4.7.A.6.a. Should one of the two H_2 or O_2 analyzers serving the drywell or suppression pool be found inoperable, the remaining analyzer of the same type serving the same compartment shall be tested for operability once per week until the defective analyzer is made operable.
7. Drywell-Suppression Chamber Differential Pressure	7. Drywell-Suppression Chamber Differential Pressure
a. Differential pressure between the drywell and suppression chamber shall be maintained at equal to or greater than 1.10 psid except as specified in (1) and (2) below:	a. The pressure differential between the drywell and suppression chamber shall be recorded at least once each shift.
(1) Within the 24-hour period subsequent to placing the reactor in the Run Mode following a shutdown, the differential shall be established. The differential may be decreased to less than 1.10 psid 24 hours prior a scheduled shutdown.	
(2) This differential may be decreased to less than 1.10 psid for a maximum of four hours during required operability testing of the HPCI system pump, the RCIC system pump, the drywell-pressure suppression chamber vacuum breakers, the suppression chamber to reactor building vacuum breakers, and to perform leak rate testing required by specification 4.7.A.2.d.4, and to allow for inerting operations to satisfy specification 3.7.A.5 requirements.	
b. If the differential pressure of specification 3.7.A.7.a cannot be maintained, and the differential pressure cannot be restored within the subsequent six (6) hour period, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown condition within the following 24 hours.	

LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT
8. If the specifications of 3.7.A.1 through 3.7.A.5 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.	
9. Purging The time which containment vent/purge valves (CV-4302, CV-4303, CV-4300, CV-4301 and CV-4307) can be open is limited to a maximum of 90 hours per calendar year, not including the 24 hour period prior to shutdown and the 24 hour period subsequent to placing the reactor in the run mode following a shutdown as specified in 3.7.A.5.b. This restriction applies whenever primary containment integrity is required.	
10. If Specification 3.7.A.9 cannot be met, prepare and submit a Special Report to the Commission pursuant to Specification 6.11.3 within the next 30 days outlining the cause of the limits being exceeded and the plans for limiting the time which these valves will be open.	

a. Reactor vessel base, weld and heat affected zone metal test specimens (Specification 4.6.A.2).

b. I-131 dose equivalent exceeding 50% of equilibrium value (Specification 4.6.B.1.h).

c. Inservice inspection (Specification 4.6.G).

d. Reactor Containment Integrated Leakage Rate Test (Specification 4.7.A.2.f).

e. Auxiliary Electrical System - Operation with inoperable components (Specification 3.8.B.4).

f. Fire Protection Systems (Specifications 3.13.A.3, 3.13.B.2, 3.13.B.3, 3.13.C.3, and 3.13.D.3).

g. Containment Vent/Purge valves (Specification 3.7.A.10).