RTS-162

TABLE 3.2-F

SURVEILLANCE INSTRUMENTATION

Minimum No. of Operable Instrument Channels	Instrument	Type Indication and Range	Action
2	Reactor Water Level	Recorder, Indicator 158"-218"	(1) (2) (3)
2	Reactor Presure	Recorder, Indicator 0-1200 psig	(1) (2) (3)
1	Drywell Pressure	Recorder, Indicator 0-80 psia	(5) (6) (3)
2	Drywell Temperature	Recorder Indicator 0-400°F	(1) (2)
2	Suppression Chamber Temperature	Recorder, Indicator 0-400°F	(1) (2) (3)
1	Suppression Chamber Water Level	Recorder -10"/0/+10" H_2O	(5) (6)
1	Control Rod Position	Process Computer, Full Travel	
1	Neutron Monitoring	SRM, IRM, LPRM 0 to 100% power	(1) (2) (3) (4)
1	Drywell Pressure	Indicator,* 0-100 psia	
1	Torus Pressure	Indicator,* 1-100 psia	

*Capable of ±0.1 psid

TABLE 4.2-F

MINIMUM TEST AND CALIBRATION FREQUENCY FOR SURVEILLANCE INSTRUMENTATION

Instrument Channel	Calibration Frequency	Instrument Check
1) Reactor Level	Once/6 months	Once Each Shift
2) Reactor Pressure	Once/6 months	Once Each Shift
3) Drywell Pressure	Once/6 months	Once Each Shift
4) Drywell Temperature	Once/6 months	Once Each Shift
5) Suppression Chamber Temperature	Once/6 months	Once Each Shift
6) Suppression Chamber Water Level	Once/6 months	Once Each Shift
7) Control Rod Position	NA	Once Each Shift
8) Neutron Monitoring	Prior to Reaching 20% Power and once per day when in Run Mode (APRM Gain Adjust when in Run Mode)	Once Each Shift (When in Startup or Run Mode)
9) Drywell Pressure	Once/Operating Cycle	Once Each Shift
10) Torus Pressure	Once/Operating Cycle	Once Each Shift

LIMITING CONDITION FOR OPERATION

must be taken out of power operation.

7. (Deleted)

8. If the specifications of 3.7.A.1 through 3.7.A.5 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 24 hours.

9. Purging

The time which containment vent/purge valves (CV-4302, CV-4303, CV-4300, CV-4301 and CV-4307) can be open is limited to a maximum of 90 hours per calendar year, not including the 24 hour period prior to shutdown and the 24 hour period subsequent to placing the reactor in the run mode following a shutdown as specified in 3.7.A.5.b. This restriction applies whenever primary containment integrity is required.

10. If Specification 3.7.A.9 cannot be met, prepare and submit a Special Report to the Commission pursuant to Specification 6.11.3 within the next 30 days outlining the cause of the limits being exceeded and the plans for limiting the time which these valves will be open.

SURVEILLANCE REQUIREMENT

functionally tested once per operating cycle in conjunction with specification 4.7.A.6.a. Should one of the two H_2 or O_2 analyzers serving the drywell or suppression pool be found inoperable, the remaining analyzer of the same type serving the same compartment shall be tested for operability once per week until the defective analyzer is made operable.

7. (Deleted)

Due to the nitrogen addition, the pressure in the containment after a LOCA could possibly increase with time. Under the worst expected conditions the containment pressure will reach 30 psig in approximately 70 days. If and when that pressure is reached, venting from the containment shall be manually initiated. The venting path will be through the Standby Gas Treatment System in order to minimize the offsite dose.

Following a LOCA, periodic operation of the drywell and torus sprays may be used to assist the natural convection and diffusion mixing of hydrogen and oxygen.

7. Standby Gas Treatment System and Secondary Containment

The secondary containment is designed to minimize any ground level release of radioactive materials which might result from a serious accident. The reactor building provides secondary containment during reactor operation, when the drywell is sealed and in service; the reactor building provides primary containment when the reactor is shut down and the drywell is

3.7.A & 4.7.A REFERENCES

1. Section 14.6 of the FSAR.

2. ASME Boiler and Pressure Vessel Code, Nuclear Vessels, Section III, maximum allowable internal pressure is 62 psig.

3. Staff Safety Evaluation of DAEC, USAEC, Directorate of Licensing, January 23, 1973.

4. 10 CFR Part 50, Appendix J, Reactor Containment Testing Requirements, Federal Register, April 19, 1976.