

NRC FORM 195
(2-76)

U.S. NUCLEAR REGULATORY COMMISSION

DOCKET NUMBER

50-331

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FILE NUMBER

TO:
Edson G. Case

FROM:
Iowa Elec Light & Power
Cedar Rapids, Iowa
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7-29-77

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DESCRIPTION

Etr notarized 8-1-77....amending Application dated 6-24-77 for Amdt of DPR-49 & Tech Specs (Appendix A to License)....trans:

ENCLOSURE

Amendment to OL/Change to Tech Specs RTS-90A consisting of a revised re-evaluation providing assurance that the most limiting break size, break location & single failure combination has been considered...w/Attached Rpt, "NEDO-21082-02-1A, July 1977, Loss of Coolant Accident Analysis Report for Duane Arnold

(see report)

40 copy encl

PLANT NAME: Duane Arnold

Retyped per LML 8-12-77

DISTRIBUTION AFTER ISSUANCE OF OPERATING LICENSE

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Notorized 08/01/77...Trans The Following

ENCLOSURE

Re submittal of Amend to Appendix A, withdrawing DPR-49 Tech Specs & subsituting Amend (RTS-90A) concerning ECCS cooling performancé & providing assurance that the most limiting break size, break location and single failure combination has been considered.

ACKNOWLEDGED

DO NOT REMOVE

2p

1/4"

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PLANT NAME: DUANE ARNOLD
jcm 08/10/77

SAFETY

FOR ACTION/INFORMATION

BRANCH CHIEF: (7) **LEAR**

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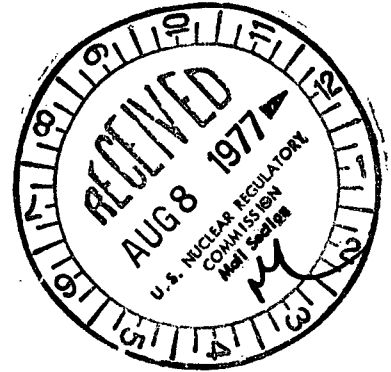
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IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office
CEDAR RAPIDS, IOWA

LEE LIU
VICE PRESIDENT - ENGINEERING

REGULATORY DOCKET FILE COPY
July 29, 1977
77-77E1453



Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulator Commission
Washington, D.C. 20545

Dear Mr. Case:

In accordance with 10CFR 50.59 and 10CFR 50.90, we transmitted our application dated June 24, 1977, for amendment of DPR-49 and the Technical Specifications (Appendix A to License) for the Duane Arnold Energy Center. We hereby amend that application with the enclosed.

Our submittal of June 24, 1977, proposed revised Limiting Average Planar Heat Generation Rates as a result of a re-evaluation of the ECCS cooling performance calculated in accordance with General Electric's evaluation model approved by the staff. This amendment (RTS-90A) provides a revised re-evaluation which provides assurance that the most limiting break size, break location and single failure combination has been considered. The conclusions of this evaluation are unchanged from the June 24, 1977, submittal.

This amendment to the application has been reviewed and approved by the DAEC Operations Committee and DAEC Safety Committee. This application does not involve significant hazards considerations. Three signed and notarized originals and thirty-seven additional copies are transmitted herewith.

772210149

Mr. Edson G. Case
IE-77-1453
Page 2

This amendment to the application, consisting of the foregoing letter and enclosure hereto, is true and accurate to the best of my knowledge and belief.

IOWA ELECTRIC LIGHT AND POWER COMPANY

BY: _____

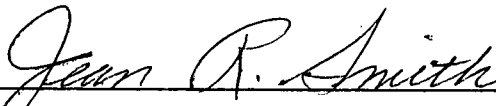


Lee Liu
Vice President, Engineering

LL/KAM/bja
Enc.

Subscribed and Sworn to before me on
this 1st day of August

cc: K. Meyer
D. Arnold
R. Lowenstein
R. Clark (NRC)
L. Root
File J-60a



Notary Public in and for the State of
Iowa.

Jean R. Smith
NOTARY PUBLIC
STATE OF IOWA
Commission Expires
September 30, 1978

PROPOSED CHANGE RTS-90A TO DAEC TECHNICAL SPECIFICATIONS

I. Affected Technical Specifications

Appendix A of the Technical Specifications for the DAEC (DPR-49) provides as follows:

Specifications 3.12 and 4.12 contain Safety Limits, Limiting Conditions for Operation and Bases which are applicable to the core thermal limits.

II. Proposed Changes in Technical Specifications

The licensees of DPR-49 propose the following changes in the Technical Specifications set forth in I above:

Change sheet 3.12-11 of the Technical Specification change submittal made on June 24, 1977 (Letter IE-77-1230; Mr. L. Liu, Vice President-Engineering, Iowa Electric Light and Power Company, to Mr. E. G. Case, Acting Director, Office of Nuclear Reactor Regulations).

III. Justification for Proposed Change

This change is submitted as a result of additional analysis on the Duane Arnold Energy Center loss-of-coolant accident performed by General Electric Company (NEDO-21082-02-1A, Class I, July 1977, Appendix A, Revision 1). This additional analysis does not affect any of the Technical Specification limits obtained in the previous submittal; it affects only the reference sheet.

IV. Review Procedure

This proposed change has been reviewed by the DAEC Operations Committee and Safety Committee which have found that this proposed change does not involve a significant hazards consideration.

3.12 REFERENCES

1. Duane Arnold Energy Center Loss-of-Coolant Accident Analysis Report, NEDO-21082-02-1A, Class I, July 1977, Appendix A, Revision 1.
2. General Electric BWR Generic Reload Application for 8 x 8 Fuel, NEDO-20360, Revision 1, November 1975.
3. "Fuel Densification Effects on General Electric Boiling Water Reactor Fuel", Supplements 6, 7 and 8, NEDM-19735, August 1973.
4. Supplement 1 to Technical Reports on Densifications of General Electric Reactor Fuels, December 14, 1973 (AEC Regulatory Staff).
5. Communication: V. A. Moore to I. S. Mitchell, "Modified GE Model for Fuel Densification", Docket 50-321, March 27, 1974.
6. R. B. Linford, Analytical Methods of Plant Transient Evaluations for the GE BWR, February 1973 (NEDO-10802).
7. General Electric Company Analytical Model for Loss-of-Coolant Analysis in Accordance with 10 CFR Part 50, Appendix K, NEDE-20566 (Draft), August 1974.
8. Duane Arnold Energy Center Reload Number Two Licensing Submittal, NEDO-21082-02, Class I, January 1977.
9. Duane Arnold Energy Center Reload Number Two Licensing Submittal, Supplement 1, Partially Drilled Core, NEDO-21082-02-01, Class I, June 1977.