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DISTRIBUTION:

Docket No. 50-331

Iowa Electric Light & Power Company ATTN: Mr. Duane Arnold Chairman of the Board & President Security Building Post Office Box 351 Cedar Rapids, Iowa 52406

Gentlemen:

A copy of a letter to Chairman Ray concerning the Advisory Committee on Reactor Safeguards' review of your application for a license to operate the Duane Arnold Energy Center is enclosed for your information.

Sincerely,

Original signed by: Voss A. Moore

Voss A. Moore, Assistant Director for Boiling Water Reactors Directorate of Licensing

Enclosure: ACRS letter, dated March 13, 1973

Newman, Reis & Axelrad 1100 Connecticut Ave., N.W., Suite 340 Washington, D.C. 20036

> Honorable Robert D. Ray Governor of Iowa Des Moines, Iowa 50319

Mr. John A. Laitner 422 Brown Iowa City, Iowa 52333

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cc: (continued)

Arnold M. Reeve, M.D. Commissioner of Public Health State Department of Health Robert Lucas Building Des Moines, Iowa 50319

Mr. Jerry Addy, Commissioner Bureau of Labor East 7th and Court Ave. Office and Laboratory Building Des Moines, Iowa 50319

Dr. George W. Brown The Anchorage, Route 3 Solon, Iowa 52333

Mr. Dudley Henderson Chairman, Linn County Board of Supervisors Cedar Rapids, Iowa 52406

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ADVISORY COMMITTEE ON REACTOR SAFEGUARDS UNITED STATES ATOMIC ENERGY COMMISSION

WASHINGTON, D.C. 20545

March 13, 1973

Honorable Dixy Lee Ray Chairman U. S. Atomic Energy Commission Washington, D. C. 20545

Subject: REPORT ON DUANE ARNOLD ENERGY CENTER

Dear Dr. Ray:

At its 155th meeting, March 8-10, 1973, the Advisory Committee on Reactor Safeguards completed its review of the application by the Iowa Electric Light and Power Company for authorization to operate the Duane Arnold Energy Center at power levels up to 1658 MWt. This project was considered at a Subcommittee meeting at the site on December 20, 1972, and at a Subcommittee meeting in Washington, D. C. on January 27, 1973. During its review the Committee had the benefit of discussions with representatives and consultants of Iowa Electric Light and Power Company, General Electric Company, Bechtel Corporation, Chicago Bridge and Iron Company, and the AEC Regulatory Staff. The Committee also had the benefit of the documents listed. The Committee reported to the Commission on the construction of this plant in its letter of December 18, 1969 and in its supplementary letter of February 11, 1970.

The Duame Arnold Energy Center Nuclear Plant will be located on a site of approximately 500 acres adjacent to the west bank of the Cedar River in a rural area approximately eight miles northwest of the city of Cedar Rapids, Iowa.

The applicant proposes to install, no later than the first scheduled refueling outage, a leak-off system intended to reduce the potential consequences of excessive leakage from the main steam isolation valves. The criteria for functional adequacy of the leak-off system and the detailed design in conformance with the criteria are not yet fully established. The Regulatory Staff should assure itself that the system finally installed does satisfy all of the considerations appropriate to the enhancement of containment reliability.

The applicant will employ a recirculation pump trip as a means of limiting the consequences of the unlikely occurrence of a failure to scrow during an anticipated transient. The trip will be installed prior to initial fuel leading. The Committee believes that this represents a substantial improvement. The specific means for implementing the pump trip should be resolved in a manner satisfactory to the Regulatory Staff.

The applicant is committed to the installation of a rod sequence control system which will render the probability of occurrence of a postulated, high-worth control rod drop accident negligibly low. This matter is under review and should be resolved in a manner satisfactory to the Regulatory Staff. Approved measures should be placed in effect prior to operation above 1% of rated power.

The shipping cask pool is physically separated from the spent fuel pool by a wall to a height above the top of stored fuel elements and a removable gate above that level. A postulated cask drop is calculated to result in penetration or cracking of the cask pool bottom if unprotected. To avoid such damage, the applicant intends to install an energy absorbing material covering the bottom of the cask pool, no later than the first refueling operation. This matter should be resolved in a manner satisfactory to the Regulatory Staff.

In the unlikely event that a break occurs in the recirculation pump discharge line, the pump impeller might act as a turbine causing the pump and motor to overspeed and become potential sources of missiles. The applicant is reviewing means of dealing with this possibility. The Committee believes that this matter should be resolved in a manner satisfactory to the Regulatory Staff.

The potential effects of some aspects of fuel performance and LOCA-related phenomena on acceptable linear fuel heat ratings for the Duane Arnold Energy Center are under study. This matter should be resolved in a manner satisfactory to the Regulatory Staff. The Committee wishes to be kept informed.

The applicant has provided protection against pipe whip in accordance with the criteria proposed by the Regulatory Staff in the Regulatory Guide, "Protection Against Pipe Whip Inside Containment", now under preparation. The Committee has emphasized the desirability of such protective measures in several letters. The Committee also recognizes that systems for restraining against pipe whip could generate undesirable stress concentrations unless properly designed and suitably installed. Therefore, particular emphasis should be devoted to the following:

(1) a better understanding of transient response in piping than is usually required; (2) quality assurance pertaining to design and installation of pipe restraints, including verification that the design computational techniques account for operational conditions and postulated transients; (3) careful examination during preoperational testing and bot startup to validate that the installation meets the design criteria.

March 13, 1973

Other problems relating to large water reactors which have been identified by the Regulatory Staff and the ACRS and cited in provious ACRS reports should be dealt with appropriately by the Regulatory Staff and the applicant as suitable approaches are developed.

The Advisory Committee on Reacter Safeguards believes that, if due regard is given to the items mentioned above, and subject to satisfactory completion of construction and preoperational testing, there is reasonable assurance that the Duana Arnold Energy Center can be operated at power levels up to 1653 MUt without undue risk to the health and safety of the public.

Sincerely yours, Original Signed by H. G. Mangelsdorf

H. C. Hangeledorf Chairman

References Attached:

References

- 1) Final Safety Analysis Report, Duane Arnold Energy Center
- 2) Amendments 1-12, Final Safety Analysis Report, Duane Arnold Energy Center
- 3) Supplement to Amendment No. 1, dated June 6, 1972
- 4) Iowa Electric Light and Power Company letter dated July 10, 1972 re: Relief Valve Discharge Line
- 5) Iowa Electric Light and Power Company letter dated October 24, 1972 re: fuel design (proprietary)
- 6) Iowa Electric Light and Power Company letter dated December 18, 1972 re: installation of a main steam line isolation valve seal system in the Duane Arnold Energy Center
- 7) Iowa Electric Light and Power Company letter dated January 15, 1973 adopts the GE NEDM-10735 "Densification Considerations in BWR Fuel Design and Performance"
- 8) Iowa Electric Light and Power Company letter dated January 16, 1973, re: the gaseous effluent discharges from the Duane Arnold Energy Center being "as low as practicable" and consistent with the proposed Appendix I to 10 CFR Part 50
- 9) Iowa Electric Light and Power Company letter dated January 22, 1973 transmitting revised operating pressure and temperature limits for Duane Arnold Energy Center
- 10) Directorate of Licensing Safety Evaluation Report dated January 23, 1973
- 11) Directorate of Licensing Supplement No. 1 to the Safety Evaluation dated March 2, 1973