

50-331

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
REPORTS FILE

TO:

NRC

FROM: IOWA ELEC LIGHT & POWER CO
CEDAR RAPIDS, IOWA

E L HAMMOND

DATE OF DOCUMENT
4-1-76

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4-5-76

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DESCRIPTION
LTR FURN CHANGE #2 TO THE SEMIANNUAL REPORT
FOR THE PERIOD JULY 1, 1975 THRU DEC 31, 1975
CONSISTING OF CHANGE TO PAGE E-6.....

ENCLOSURE

ACKNOWLEDGED

DO NOT REMOVE

LIC. ASST.(E)...LTR & MAIL CONTROL ONLY
NOTE: DOCKET CLERK WILL SEND EPA(2), F & W
(2), F & D & NOAA CYS TO ENVIRO LIC. ASST.
FOR DISTRIBUTION....TOTAL (6) CYS

PLANT NAME: **DWANE ARNOLD**

SAFETY

FOR ACTION/INFORMATION

ENVIRO 4-7-76 RB

BRANCH CHIEF: **LEAR**
W/ 1 CYS FOR ACTION

INTERNAL DISTRIBUTION

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 I & E (2)
 MIPC (4)
 HANAUER
 EISENHUT
 LIC. ASST.(L) LTR & MAIL CONTROL ONLY **PARRISH**
 DENTON

EXTERNAL DISTRIBUTION

LPDR: CEDAR RAPIDS, IO
 TIC
 NSIC
ACRS
W/ CYS 16 ~~HOLDING~~ SENT TO LA

CONTROL NUMBER

P
3446

IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER
P. O. Box 351
Cedar Rapids, Iowa 52406

April 1, 1976
DAEC - 76- 83



Regulatory Docket **E-6**

Director, Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
1717 H Street N.W.
Washington, D.C. 20545

Subject: Revision 2 to Semiannual Operating Report
for the Duane Arnold Energy Center

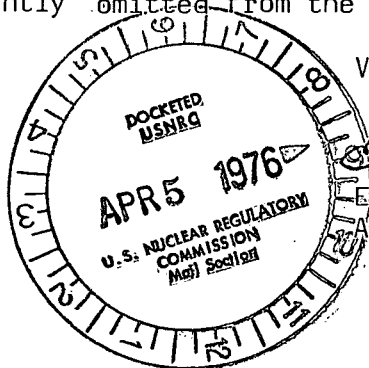
File: A-118e, A-104

Dear Sir:

Please find enclosed 40 copies of Revision 2 to the Duane Arnold Energy Center Semiannual Report for the period July 1, 1975 through December 31, 1975. Revision 2 consists of a change to page E-6.

Page E-6 was revised to include a facility design change that was inadvertently omitted from the original report.

Very truly yours,



Ellery L. Hammond
Ellery L. Hammond
Assistant Chief Engineer

DLW/ELH/mg
Enclosure

cc: D. Arnold w/o enclosures
J. Wallace "
G. Hunt "
E. Hammond "
D. Wilson "

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DCR #462 MAIN STEAM

Description of Change

Modification of relief valve piping supports inside torus to include the reinforcement of the existing structural steel components.

Safety Evaluation

The intent of this design change is to increase the margin of safety provided by the relief valve discharge piping restraint system inside the suppression chamber.

An analysis has been conducted to substantiate the inclusion of new structural material and strengthening of existing material, with respect to the structural integrity of the torus and it's design intent. This design change does not present hazards or considerations not already addressed in the Safety Analysis Report.

DCR #464 STANDBY GAS TREATMENT

Description of Change

Replace the 25 amp circuit breakers for heaters EC-5805 A & B with 30 AMP breakers.

Safety Evaluation

This change does not have any effect on plant safety, and in no way alters the original safety analysis.