

B04/24/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-331

REC: KEPPLER J G
NRC

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IA ELEC LIGHT & PWR

DOC DATE: 04/17/78
DATE RCVD: 04/21/78

DOCTYPE: LETTER NOTARIZED: NO

COPIES RECEIVED

SUBJECT: LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-331/78-018) ON 04/03/78 CONCERNING DURING ANNUAL SURVEILLANCE TESTING OF SAFETY/ RELIEF VALVES, ONE SAFETY VALVE AND THREE RELIEF VALVES WERE FOUND TO HAVE OUT OF SPECIFICATION AS FOUND SETPOINTS. . . W/ATT.

PLANT NAME: DUANE ARNOLD

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: *w*

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

FOR ACTION: BR CHIEF LEAR**W/4 ENCL

INTERNAL: REG FILE**W/ENCL
*J & E**W/2 ENCL*
SCHRÖEDER/IPPOLITO**W/ENCL
NOVAK/CHECK**W/ENCL
KNIGHT**W/ENCL
HANAUER**W/ENCL
EISENHUT**W/ENCL
SHAD**W/ENCL
KREGER/J. COLLINS**W/ENCL
K SEYFRIT/IE**W/ENCL

NRC PDR**W/ENCL
MIPC**W/3 ENCL
HOUSTON**W/ENCL
EEB**W/ENCL
BUTLER**W/ENCL
TEDESCO**W/ENCL
BAER**W/ENCL
VOLLMER/BUNCH**W/ENCL
ROSA**W/ENCL

EXTERNAL: LPDR'S
CEDAR RAPIDS, IA**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

COPIES NOT SUBMITTED PER
REGULATORY GUIDE 10.1

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+2P

CONTROL NBR: 781140019

***** THE END ***** *60*

IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER
P. O. Box 351
Cedar Rapids, Iowa 52406
April 17, 1978
DAEC - 78 - 220

Mr. James G. Keppler, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission-Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Subject: Licensee Event Report No. 78-018
(14 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

Ellery L. Hammond
Ellery L. Hammond
Chief Engineer
Duane Arnold Energy Center

Docket 50-331

attachment

ELH/JVS/nf

cc: Director, Office of Inspection and Enforcement (40)
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director, Management Information and Program Control (3)
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

APR 19 1978

LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I A D A C 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 1 4 5

CON'T 0 1 REPORT SOURCE L 6 0 5 0 0 0 3 3 1 7 0 4 0 3 7 8 8 0 4 1 7 7 8 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 During annual surveillance testing of safety/relief valves, one safety v
0 3 alve and three relief valves were found to have out of specification as
0 4 found setpoints. The S/V was 9 PSI high, one R/V 93 PSI high, one R/V 4
0 5 PSI high and one R/V 4 PSI low. Setpoint requirements listed in T.S. Sec
0 6 tion 2.2.1. NSSS Vendor analysis of as found setpoints predicted the tra
0 7 nsient peak pressure would increase by 5 PSI which would not degrade the
0 8 vessel overpressure protection capability.

0 9 SYSTEM CODE S F 11 CAUSE CODE E 12 CAUSE SUBCODE B 13 COMPONENT CODE V A L V E X 14 COMP. SUBCODE P 15 VALVE SUBCODE B 16
17 LER/RO REPORT NUMBER 7 8 EVENT YEAR 7 8 SEQUENTIAL REPORT NO. 0 1 8 OCCURRENCE CODE 0 1 REPORT TYPE T REVISION NO. 0
ACTION TAKEN B 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 ATTACHMENT SUBMITTED Y 23 NPRD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER L 25 COMPONENT MANUFACTURER T 0 2 0

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 Cause unknown. Vendor and test facility representative investigations di
1 1 d not produce a definitive cause for the out of specification setpoints.
1 2 The pilot assemblies were cleaned and reworked as required and retested
1 3 satisfactorily.

1 4 FACILITY STATUS H 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Surveillance Test 32
1 5 ACTIVITY CONTENT Z 33 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36
1 6 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39
1 7 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41
1 8 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43
1 9 PUBLICITY ISSUED N 44 DESCRIPTION NA 45

DUANE ARNOLD ENERGY CENTER

Iowa Electric Light and Power Company

LICENSEE EVENT REPORT-Supplemental Data

Docket Number 050-0331

Licensee Event Report Date: April 17, 1978

Reportable Occurrence No: 78-018

Event Description

During annual surveillance testing of safety/relief valves, one safety valve and three relief valves were found to have out of specification as-found setpoints. All six main steam relief valves and one safety valve were tested. The following is a tabulation of the as-found test results:

<u>Valve</u>	<u>Serial No.</u>	<u>Required Setpoint</u>	<u>As-Found Setpoint</u>
S/V	8491	1240 ± 12 PSI	1261 PSI
R/V	189	1100 ± 11 PSI	1204 PSI
R/V	227	1110 ± 11 PSI	1125 PSI
R/V	218	1090 ± 11 PSI	1075 PSI
R/V	199	1100 ± 11 PSI	1104 PSI
R/V	176	1110 ± 11 PSI	1104 PSI
R/V	226	1090 ± 11 PSI	1099 PSI

With the as-found setpoints for all six relief valves, the NSSS Vendor conducted an analysis to determine the expected transient pressure peak. The results of this analysis indicated the peak was predicted to increase by approximately 5 PSI which would not degrade the vessel overpressure protection capability.

Cause Description

Unknown. Investigation by Vendor representatives and test facility personnel did not produce a definitive cause for the out of specification setpoints.

Corrective Action

Corrective actions taken were as follows:

- S/V # 8491 - The main disk and seat were lapped and the valve cleaned and retested satisfactorily.
- R/V #189 - The next test run following the initial as-found test produced a setpoint of 1114 PSI with no adjustments or work of any kind done. The pilot assembly was disassembled and inspected and no cause for the high as-found setpoint was positively identified. The pilot and second stage disk were lapped. The pilot assembly was cleaned and reassembled and tested satisfactorily.
- R/V #227 - During subsequent testing this pilot assembly demonstrated repeatability within allowable tolerance and no rework or further testing were required.

R/V #218

The pilot assembly was disassembled and inspected, cleaned, the pilot and second stage disks lapped, and the pilot reassembled and tested satisfactorily.