

50-331

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FILE NUMBER

INCIDENT REPORT

TO: James G. Keppler

FROM: Iowa Electric Light Co.
Cedar Rapids, Iowa 52406
Ellery L. Hammond

DATE OF DOCUMENT
10/11/77

DATE RECEIVED
10/20/77

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DESCRIPTION

PLANT NAME: Duane Arnold

1p

MDC
10/21/77

ENCLOSURE

Licensee Event Report 50-331-77-78) concerning damage noted on HPCI to RHR piping insulation.

2p

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED
SEND DIRECTLY TO KREGER/J. COLLINS

1 cy ENCL Rec'd *

FOR ACTION/INFORMATION

BRANCH CHIEF:	LEAR
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CONTROL NUMBER

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IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER
P. O. Box 351
Cedar Rapids, Iowa 52406

October 11, 1977
DAEC-77 - 511



Mr. James G. Keppler, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission-Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Subject: Licensee Event Report No. 77-78
(14 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

Ellery L. Hammond

Ellery L. Hammond
Chief Engineer
Duane Arnold Energy Center

ELH/JVS/mg
Docket 50-331
attachment

cc: Director, Office of Inspection and Enforcement (40)
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Director, Management Information and Program Control (3)
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

OCT 13 1977
772940037

DUANE ARNOLD ENERGY CENTER

Iowa Electric Light and Power Company

LICENSEE EVENT REPORT-Supplemental Data

Licensee Event Report Date: 101077

Reportable Occurrence No: 77-78

DESCRIPTION OF OCCURRENCE

While conducting an unrelated engineering review of seismic snubber anchors, it was determined that a short section of RHR heat exchanger inlet piping had been overstressed. This determination was based on calculated pipe movement indicated by observed piping insulation damage. ND examinations revealed no apparent defects in the affected piping.

CAUSE OF OCCURRENCE

The cause of this occurrence could not be positively identified. The most probable explanation is that the movement and resulting stress occurred during a hydraulic shock to the HPCI steam supply piping which occurred on 6/11/74. This event was documented in AO 74-11. The RHR heat exchanger steam inlet piping (steam condensing mode) is tied to the HPCI steam supply line and thus is affected by movement in this line. No shocks have occurred in the HPCI steam line since 6/11/74 which would have produced the piping insulation damage noted.

CORRECTIVE ACTION

All areas that were calculated to have been overstressed were inspected by non-destructive examination with no apparent defects found. See also the corrective actions noted in AO Report 74-11.

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