To							
NRC FORM 195 (2-76)	-		U.S. NU	CLEA	R REGULATORY COMMISSION	DO	скет NUMBER 50-331
	NFOR P	ART 50 DOCKET	MAI	FERIAL	FILE NUMBER ÎNCIDENT REPORT		
TO.			FROM:			DA	TE OF DOCUMENT
TO:				& 1	Power Compa <b>n</b> y	8/	/9/77
Mr. James G. Keppler			Cedar Rapi				
			Ellery L.	Ham	nond		TE RECEIVED 8/24/77
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Duane Arnold RJL 8/25/77 (1-P)							
		•			NOTE: IF PERSONNEL SEND DIRECTL	EXF Y T(	POSURE IS INVOLVED ) KREGER/J. COLLINS
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Regulatory

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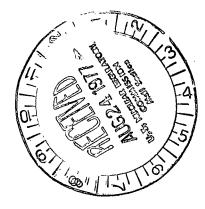
## IOWA ELECTRIC LIGHT AND POWER COMPANY

File Cyd

DUANE ARNOLD ENERGY CENTER P. O. Box 351 Cedar Rapids, Iowa 52406 August 9, 1977

DAEC -77 - 413

Mr. James G. Keppler, Director Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission-Region III 799 Roosevelt Road Glen Ellyn, Illinois 60137



Subject: Licensee Event Report No. 77-60 (14 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

Ellery L. Hammond / Buy

Ellery L. Hammond Chief Engineer Duane Arnold Energy Center

Docket 50-331

attachment

ELH/JVS/mg

cc: Director, Office of Inspection and Enforcement (40) U. S. Nuclear Regulatory Commission Washington, D.C. 20555 -

Director, Management Information and Program Control (3) U. S. Nuclear Regulatory Commission Washington, D.C. 20555

> AUG 1 1 1977 772370093

*CONTROL BLOCK:
LICENSEE NAME 01 I A D A C I 0 0 0 0 0 0 0 0 4 1 1 1 1 0 1 7 B 9 14 15 25 26 30 31 32
CATEGORY       REPORT TYPE       REPORT SOURCE       OOCKET NUMBER       EVENT OATE       REPORT OATE         01 CON'T       1       1       0       5       0       -       0       3       3       1       0       7       2       6       7       7       0       8       0       9       7       7         7       6       57       58       59       60       61       66       69       74       75       80
EVENT DESCRIPTION  OE Louring the performance of a design study conducted as a result of a non
7 89 03 conformance review concerning reactor high pressure recirculation pump
7 69 04 trip instrumentation, the trip setting listed in Table 3.2-6 of Technic
7 8 9 05 al Specifications was found to be in error. The trip setting specified 60
7       8       9         0.6
7     8     9     PRIME COMPONENT     COMPONENT     COMPONENT     60       SYSTEM CODE     COMPONENT CODE     COMPONENT     COMPONENT     MANUFACTURER     VIOLATION       07     I     A     F     Z     Z     Z     Z     Z     N       7     8     9     10     11     12     17     43     44     47     48
CAUSE OESCRIPTION
7 8 9 09 tially developed. A Technical Specification change will be submitted to
7 8 9 10 correct the required trip setting.
7     8     9     FACILITY     STATUS     METHOD OF     80       STATUS     % POWER     OTHER STATUS     DISCOVERY     DISCOVERY     DISCOVERY     0       11     -E     0     9     4     NA     C     NA
7     8     9     10     12     13     44     45     46     80       FORM OF ACTIVITY RELEASED     CONTENT OF RELEASE     AMOUNT OF ACTIVITY NA     LOCATION OF RELEASE NA     LOCATION OF RELEASE NA     10     11     10     11     10 <td< td=""></td<>
NUMBER         TYPE         DESCRIPTION           13         0         0         Z         NA           7         8         9         11         12         13
PERSONNEL INJURIES         B0           14         0
OFFSITE CONSEQUENCES       15     NA
LOSS OR DAMAGE TO FACILITY TYPE DESCRIPTION
7 8 9 10 PUBLICITY 80
17     NA       7     8 9
ADOITIONAL FACTORS          18       Event Desc. Cont than or equal to 1120 PSIG. The applicable surveilla:       1         7       8       9
nce test has been modified to reflect the proper setting, (RO 77-60)
NAME:J. Van SickelPHONE:319-851-5611B0

CPC



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