To							
NRC FORM 195 (2-76)	-		U.S. NU	CLEA	R REGULATORY COMMISSION	DO	скет NUMBER 50-331
	NFOR P	ART 50 DOCKET	MAI	FERIAL	FILE NUMBER ÎNCIDENT REPORT		
TO.			FROM:			DA	TE OF DOCUMENT
TO:				& 1	Power Compa n y	8/	/9/77
Mr. James G. Keppler			Cedar Rapi				
			Ellery L.	Ham	nond		TE RECEIVED 8/24/77
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							SIGNED CY RECEIVED
DESCRIPTION			_, I	ENC	LOSURE		
PLANT NAME:					Licensee Event Repor 7/26/77 concerning t in table 3.2-6 of te be in error	he ch	trip setting listed specs being found to
Duane Arnold RJL 8/25/77 (1-P)							
		•			NOTE: IF PERSONNEL SEND DIRECTL	EXF Y T(POSURE IS INVOLVED) KREGER/J. COLLINS
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Regulatory

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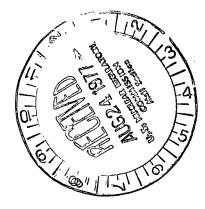
IOWA ELECTRIC LIGHT AND POWER COMPANY

File Cyd

DUANE ARNOLD ENERGY CENTER P. O. Box 351 Cedar Rapids, Iowa 52406 August 9, 1977

DAEC -77 - 413

Mr. James G. Keppler, Director Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission-Region III 799 Roosevelt Road Glen Ellyn, Illinois 60137



Subject: Licensee Event Report No. 77-60 (14 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

Ellery L. Hammond / Buy

Ellery L. Hammond Chief Engineer Duane Arnold Energy Center

Docket 50-331

attachment

ELH/JVS/mg

cc: Director, Office of Inspection and Enforcement (40) U. S. Nuclear Regulatory Commission Washington, D.C. 20555 -

Director, Management Information and Program Control (3) U. S. Nuclear Regulatory Commission Washington, D.C. 20555

> AUG 1 1 1977 772370093

*CONTROL BLOCK:
LICENSEE NAME 01 I A D A C I 0 0 0 0 0 0 0 0 4 1 1 1 1 0 1 7 B 9 14 15 25 26 30 31 32
CATEGORY REPORT TYPE REPORT SOURCE OOCKET NUMBER EVENT OATE REPORT OATE 01 CON'T 1 1 0 5 0 - 0 3 3 1 0 7 2 6 7 7 0 8 0 9 7 7 7 6 57 58 59 60 61 66 69 74 75 80
EVENT DESCRIPTION OE Louring the performance of a design study conducted as a result of a non
7 89 03 conformance review concerning reactor high pressure recirculation pump
7 69 04 trip instrumentation, the trip setting listed in Table 3.2-6 of Technic
7 8 9 05 al Specifications was found to be in error. The trip setting specified 60
7 8 9 0.6
7 8 9 PRIME COMPONENT COMPONENT COMPONENT 60 SYSTEM CODE COMPONENT CODE COMPONENT COMPONENT MANUFACTURER VIOLATION 07 I A F Z Z Z Z Z N 7 8 9 10 11 12 17 43 44 47 48
CAUSE OESCRIPTION
7 8 9 09 tially developed. A Technical Specification change will be submitted to
7 8 9 10 correct the required trip setting.
7 8 9 FACILITY STATUS METHOD OF 80 STATUS % POWER OTHER STATUS DISCOVERY DISCOVERY DISCOVERY 0 11 -E 0 9 4 NA C NA
7 8 9 10 12 13 44 45 46 80 FORM OF ACTIVITY RELEASED CONTENT OF RELEASE AMOUNT OF ACTIVITY NA LOCATION OF RELEASE NA LOCATION OF RELEASE NA 10 11 10 11 10 <td< td=""></td<>
NUMBER TYPE DESCRIPTION 13 0 0 Z NA 7 8 9 11 12 13
PERSONNEL INJURIES B0 14 0
OFFSITE CONSEQUENCES 15 NA
LOSS OR DAMAGE TO FACILITY TYPE DESCRIPTION
7 8 9 10 PUBLICITY 80
17 NA 7 8 9
ADOITIONAL FACTORS 18 Event Desc. Cont than or equal to 1120 PSIG. The applicable surveilla: 1 7 8 9
nce test has been modified to reflect the proper setting, (RO 77-60)
NAME:J. Van SickelPHONE:319-851-5611B0

CPC



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