

50-331

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER  
INCIDENT REPORT

TO: Mr. J. G. Keppler

FROM: Iowa Elec Light & Power Co  
Cedar Rapids, Iowa 52406  
E. L. Hammond

DATE OF DOCUMENT  
3-18-77

DATE RECEIVED  
4-12-77

LETTER  
 ORIGINAL  
 COPY

NOTORIZED  
 UNCLASSIFIED

PROP INPUT FORM

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1 SIGNED

DESCRIPTION Ltr furn notification of reportable occurrence involving MSIV 4413 was found to have a leak rate of 36.70 SCFH & advising that after tightening the valve the leakage reduced...

ENCLOSURE

ACKNOWLEDGED

DO NOT REMOVE

PLANT NAME: Duane Arnold Plant

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

DHL

FOR ACTION/INFORMATION

BRANCH CHIEF:	<i>Lear</i>
W/3 CYS FOR ACTION	
LIC. ASST.:	<i>Parrish</i>
W/1 CYS	
ACRS 16 CYS HOLDING/SENT	<i>As CAT B</i>

INTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> REG FILE					
NRC PDR					
I & E (2)					
MIPC					
SCHROEDER/IPPOLITO					
HOUSTON					
NOVAK/CHECK					
GRIMES					
CASE					
BUTLER					
HANAUER					
TEDESCO/MACCARY					
EISENHUT					
BAER					
SHAO					
VOLLMER/BUNCH					
KREGER/J. COLLINS					

EXTERNAL DISTRIBUTION

LPDR: Cedar Rapids, IA					
TIC:					
NSIC:					

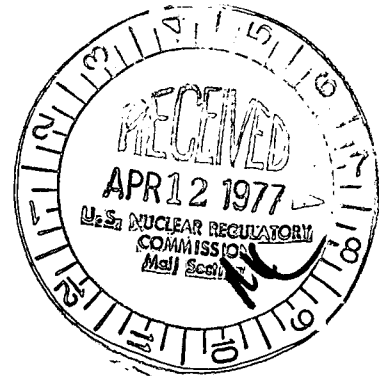
CONTROL NUMBER

771050223 60

*Curran*

# IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER  
P. O. Box 351  
Cedar Rapids, Iowa 52406  
March 18, 1977  
DAEC -77 - 177



Mr. James G. Keppler, Director  
Office of Inspection and Enforcement-Region III  
U. S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

File CY-

Subject: Prompt Notification of a Reportable Occurrence

Regulatory

File: A-118a

Dear Mr. Keppler:

Per the request of Mr. H. Kister of your office, this letter is notification of a Reportable Occurrence at the Duane Arnold Energy Center on March 14, 1977.

During Type C Leak Rate Testing of the Main Steam Isolation Valves, MSIV 4413 was found to have a leak rate of 36.70 SCFH. The Technical Specification limit is 11.5 SCFH. It should be noted that after tightening of the valve stem packing, the leakage was reduced to 5.24 SCFH indicating that the majority of the leakage was through the packing and not across the valve seat. It is our opinion that the Technical Specification limit of 11.5 SCFH is intended as a seat leakage criteria and that the 5.24 SCFH leak rate after the valve stem packing was tightened indicates that the as found valve leak rate was within Technical Specification limits. We plan to initiate a Technical Specification change to clarify the flow path for the 11.5 SCFH Technical Specification criteria.

Mr. Charlie Brown of your office was advised of this occurrence during his site inspection of March 14 and 15, 1977.

Very truly yours,

*Ellery L. Hammond*  
Ellery L. Hammond  
Chief Engineer, DAEC

ELH/DLW/mg  
77-21

Director, Management Information and Program Control (2)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

MAR 23 1977

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