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Mr. James G. Keppler		Iowa Electric Light & Power Co. Cedar Rapids, Iowa Ellery L. Hammond			DATE RECEIVED			
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Ltr. trans the following				Licensee Event Report (RO 50-331/77-22) on 3/22/77 concerning a hairline crack in the pipe to elbow weld in the discharge bypass				
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IOWA ELECTRIC LIGHT AND POWER COMPANY

Regulatory

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DUANE ARNOLD ENERGY CENTER P. O. Box 351 Cedar Rapids, Iowa 52406 April 5, 1977 DAEC - 77 -207

Mr. James G. Keppler, Director Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission-Region III 799 Roosevelt Road Glen Ellyn, Illinois 60137

> Subject: Licensee Event Report No. 77-22 (14 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

El Hammend 1

Ellery L. Hammond Chief Engineer Duane Arnold Energy Center

ELH/JVS/mg

Docket 50-331

attachment

cc: (Director, Office of Inspection and Enforcement)(40)
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

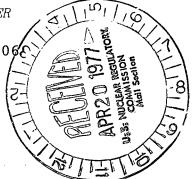
Director, Management Information and Program Control (3) U. S. Nuclear Regulatory Commission Washington, D.C. 20555

LICENSEE EVEN. HEPOHI PRINT ALL REQUIRED INFORMATION) **TPLEN** CONTROL BLOCK: EVENT LICENSE LICENSEE TYPE TYPE LICENSE NUMBER NAME 1 1 0 0 | Ţ |0|DA 0 Q IA С 1 32 31 25 26 REPORT DATE REPORT REPORT EVENT DATE DOCKET NUMBER-CATEGORY SOURCE 04 3 1 1 2 0 3 2 7 7 7 7 0 0 5 0 - 0 3 2 Ť LI O 1 CON'T RÔ 68 69 59 60 6 57 58 EVENT DESCRIPTION During inservice inspection ultrasonic examination indicated a possible 0 5 80 hairline crack in the pipe to elbow weld in the discharge bypass pipe 8 9 03 80 old of the "A" reactor recirculation loop. The indication was a possible cr 80 05 ack 1/2 inch to 1 inch in length and less than 5 percent pipe wall thick 80 (RO 77-22) Repairs commenced 4/5/77. ness in depth. 06 80 ัย 9 PRIME COMPONENT COMPONENT SYSTEM CAUSE CODE VIOLATION SUPPLIER MANUFACTURER COMPONENT CODE Z|9 |9 |9 Y | Z | X I P E X C B F P 07 48 43 12 89 10 CAUSE DESCRIPTION Action has commenced to verify that the crack does exist and t Unknown. 08 80 89 o return the weld to a condition which meets the criteria of the origin 09 80 8 9 Supplemental report to be submitted. al acceptance standard. 10 80 89 METHOD OF FACILITY DISCOVERY DESCRIPTION OTHER STATUS DISCOVERY % POWER STATUS Inservice Inspection B $\boxed{1}$ NA |H | 80 45 46 12 13 ۵۵ ١Ō 9 FORM DF CONTENT ACTIVITY LOCATION OF RELEASE AMOUNT OF ACTIVITY RELEASED OF RELEASE NA 12 NA Z Z 80 45 9 10 11 8 PERSONNEL EXPOSURES **OESCRIPTION** NUMBER TYPE NA 13 000 Ζ 80 8 9 12 13 PERSONNEL INJURIES NUMBER DESCRIPTION NA 0 0 0 14 80 8 9 11 12 OFFSITE CONSEQUENCES NA 1 5 80 R q LOSS OR DAMAGE TO FACILITY **DESCRIPTION** TYPE NA 16 L 80 10 9 PUBLICITY NA 17 80 8.8 ADDITIONAL FACTORS NA 18 80 8 9 19 80 89 J. Van Sickel **319-851-**5611 PHONE: NAME: GP O 881-667

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IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER P. O. Box 351 Cedar Rapids, Iowa 52406 April 5, 1977 DAEC - 77 -207



Mr. James G. Keppler, Director Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission-Region III 799 Roosevelt Road Glen Ellyn, Illinois 60137

> Subject: Licensee Event Report No. 77-22 (14 day)

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Very truly yours,

ElHammend

Ellery L. Hammond Chief Engineer Duane Arnold Energy Center

ELH/JVS/mg

Docket 50-331

attachment

cc:

Director, Office of Inspection and Enforcement (40) U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Director, Management Information and Program Control (3) U. S. Nuclear Regulatory Commission Washington, D.C. 20555

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	CONTROL BLOCK	(PLEASE PRINT ALL REQUIRED INFORMATION)
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03	hairline crack in the pipe to elbow weld	in the discharge bypass pipe
7 E	9 of the "A" reactor recirculation loop. T	he indication was a possible cr
7 B	9 ack 1/2 inch to 1 inch in length and less	than 5 percent pipe wall thick 1
7		(RO 77-22) 60
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7 8	9 al acceptance standard. Supplemental repo	ort to be submitted.
7 8	9 FACILITY	THOD DF 80
	STATUS % POWER OTHER STATUS DIS H [] [] [] NA] 9 10 12 13 44	COVERY DISCOVERY DESCRIPTION B Inservice Inspection 45 46
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16		1
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