DSB

Iowa Electric Light and Power Company
September 27, 1979
DAEC-79-254

Mr. James G. Keppler, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission - Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Subject: Licensee Event Report No. 79-020
(30 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

EL Hammond /lh

Ellery L. Hammond
Chief Engineer
Duane Arnold Energy Center

Docket 50-331

attachment
ELH/JVS/lh

cc: Director, Office of Inspection and Enforcement (30)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Director, Management Information and Program Control (3)
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

7910030465

OCT 1 19[illegible]

NRC FORM 366

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
01	LICENSEE CODE (2)	I A D A C 1
	LICENSE NUMBER (3)	0 0 - 0 0 0 0 0 - 0 0
	LICENSE TYPE (4)	4 1 1 1 1
	CAT (5)	
CON'T 01	REPORT SOURCE (6)	L
	DOCKET NUMBER (7)	0 5 0 0 0 3 3 1
	EVENT DATE (8)	0 9 0 2 7 9
	REPORT DATE (9)	0 9 2 7 7 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 Following a power increase in accordance with approved procedures, CPR
03 was found to be 1.289, below the Technical Specification limit of greate
04 r than or equal to 1.290. Recirculation flow was immediately reduced and
05 22 minutes later CPR was again checked and found to be 1.303, well above
06 the Tech. Spec. limit. Reference Tech. Spec. Table 3.12-2.
07
08

SYSTEM CODE (11)	CAUSE CODE (12)	CAUSE SUBCODE (13)	COMPONENT CODE (14)	COMP. SUBCODE (15)	VALVE SUBCODE (16)
R B	X	Z	X X X X X X	Z	Z

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
7 9	—	0 2 0	/	0 3	L	—	0

ACTION TAKEN (18)	FUTURE ACTION (19)	EFFECT ON PLANT (20)	SHUTDOWN METHOD (21)	HOURS (22)	ATTACHMENT SUBMITTED (23)	NPRD-4 FORM SUB. (24)	PRIME COMP. SUPPLIER (25)	COMPONENT MANUFACTURER
X	Z	Z	Z	0 0 0 0	N	N	Z	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 This occurrence was the result of a Xenon transient causing thermal powe
11 r to increase. Corrective actions to increase CPR were proper and well
12 within the time limits specified in Technical Specifications Paragraph
13 3.12-C. The plant has been and will continue to be operated in a manner
14 which will minimize this type of occurrence.

Line	FACILITY STATUS (28)	% POWER (29)	OTHER STATUS (30)	METHOD OF DISCOVERY (31)	DISCOVERY DESCRIPTION (32)
15	F	0 9 6	NA	A	Operator Observation

Line	ACTIVITY RELEASED (33)	CONTENT OF RELEASE (34)	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z	Z	NA	NA

PERSONNEL EXPOSURES

Line	NUMBER (37)	TYPE (38)	DESCRIPTION (39)
17	0 0 0	Z	NA

PERSONNEL INJURIES

Line	NUMBER (40)	DESCRIPTION (41)
18	0 0 0	NA

LOSS OF OR DAMAGE TO FACILITY (43)

Line	TYPE (42)	DESCRIPTION
19	Z	NA

PUBLICITY

Line	ISSUED (44)	DESCRIPTION (45)	NRC USE ONLY
20	N	NA	