

50-331

## NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER

INCIDENT REPORT

TO:

MR J G KEPPLER

FROM IOWA ELEC LIGHT & POWER CO  
CEDAR RAPIDS, IOWA  
g g hunt

DATE OF DOCUMENT

6-10-76

DATE RECEIVED

6-14-76

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1 Sign

## DESCRIPTION

LTR TRANS THE FOLLOWING.....

## ENCLOSURE

LICENSEE EVENT REPORT (RO 76-37) 50-331/  
ON 6-1-76 RE MCPR REACHED 1.759 AND ONE  
REACHED 1.689 BOTH EXCEEDING TECH SPEC....

DO NOT REMOVE

ACKNOWLEDGED

PLANT NAME: DUANE ARNOLD

1 PG  
4 enclNOTE: IF PERSONNEL EXPOSURE IS INVOLVED  
SEND DIRECTLY TO KREGER/J. COLLINS

## SAFETY

## FOR ACTION/INFORMATION

## ENVIRO

6-15-76 RB

X	BRANCH CHIEF:	G. LEAR
	W/3 CYS FOR ACTION	
X	LIC. ASST:	C. PARRISH
	W/ CYS	
	ACRS CYS HOLDING/SENT TO LA	

## INTERNAL DISTRIBUTION

<input checked="" type="checkbox"/>	REG FILE	
<input checked="" type="checkbox"/>	NRC PDR	
<input checked="" type="checkbox"/>	I & E (2)	
<input checked="" type="checkbox"/>	MIPC (3)	
<input checked="" type="checkbox"/>	SCHROEDER/IPPOLITO	
<input checked="" type="checkbox"/>	HOUSTON	
<input checked="" type="checkbox"/>	NOVAK/CHECK	
<input checked="" type="checkbox"/>	GRIMES/SCHWENCER	1 EACH
<input checked="" type="checkbox"/>	CASE	
<input checked="" type="checkbox"/>	HANAUER	
<input checked="" type="checkbox"/>	TEDESCO/MACCARY	
<input checked="" type="checkbox"/>	EISENHUT	
<input checked="" type="checkbox"/>	BAER	
<input checked="" type="checkbox"/>	SHAO	
<input checked="" type="checkbox"/>	VOLLMER/BUNCH	
<input checked="" type="checkbox"/>	KREGER/J. COLLINS	

## EXTERNAL DISTRIBUTION

## CONTROL NUMBER

<input checked="" type="checkbox"/>	LPDR: CEDAR RAPIDS, IO
<input checked="" type="checkbox"/>	TIC
<input checked="" type="checkbox"/>	NSIC

6042

# IOWA ELECTRIC LIGHT AND POWER COMPANY

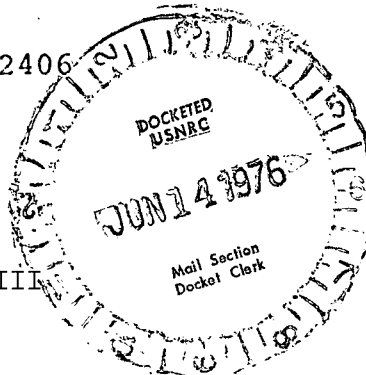
DUANE ARNOLD ENERGY CENTER

P. O. Box 351

Cedar Rapids, Iowa 52406

June 10, 1976

DAEC -76 -189



Regulatory Docket File

Mr. James G. Keppler, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission - Region III  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Subject: Licensee Event Report No. 76-37  
(30 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

*G. G. Hunt*

G. G. Hunt  
Chief Engineer  
Duane Arnold Energy Center



Docket 50-331

attachment

GGH/DLW/mg

cc: ( Director, Office of Inspection and Enforcement ) (30)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Director, Management Information and Program Control (3)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

6042

# LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME						LICENSE NUMBER										LICENSE TYPE					EVENT TYPE				
<span style="border: 1px solid black; padding: 2px;">01</span>	<span style="border: 1px solid black; padding: 2px;">I</span>	<span style="border: 1px solid black; padding: 2px;">A</span>	<span style="border: 1px solid black; padding: 2px;">D</span>	<span style="border: 1px solid black; padding: 2px;">A</span>	<span style="border: 1px solid black; padding: 2px;">C</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">-</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">-</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">4</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">3</span>		
7	8	9				14	15										25	26				30	31	32	
CONT		CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER					EVENT DATE					REPORT DATE									
<span style="border: 1px solid black; padding: 2px;">01</span>		<span style="border: 1px solid black; padding: 2px;">57</span>	<span style="border: 1px solid black; padding: 2px;">58</span>	<span style="border: 1px solid black; padding: 2px;">L</span>	<span style="border: 1px solid black; padding: 2px;">L</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">5</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">-</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">3</span>	<span style="border: 1px solid black; padding: 2px;">3</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">6</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">7</span>	<span style="border: 1px solid black; padding: 2px;">6</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">6</span>	<span style="border: 1px solid black; padding: 2px;">1</span>	<span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">7</span>	<span style="border: 1px solid black; padding: 2px;">6</span>
7	8			59	60	61						68		69							74	75			80

## EVENT DESCRIPTION

<span style="border: 1px solid black; padding: 2px;">02</span>	During control rod withdrawals, two MCPR violations occurred. First	80
<span style="border: 1px solid black; padding: 2px;">03</span>	one, MCPR reached 1.759 (limit 1.761); second one, MCPR reached 1.689	80
<span style="border: 1px solid black; padding: 2px;">04</span>	(limit 1.717). Group 42 rods inserted one notch in both cases to re-	80
<span style="border: 1px solid black; padding: 2px;">05</span>	turn MCPR within limits. (RO 76-37).	80
<span style="border: 1px solid black; padding: 2px;">06</span>		80

SYSTEM CODE	CAUSE CODE	COMPONENT CODE					PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER			VIOLATION	
<span style="border: 1px solid black; padding: 2px;">07</span>	<span style="border: 1px solid black; padding: 2px;">R</span> <span style="border: 1px solid black; padding: 2px;">C</span>	<span style="border: 1px solid black; padding: 2px;">D</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	<span style="border: 1px solid black; padding: 2px;">9</span>	<span style="border: 1px solid black; padding: 2px;">9</span>	<span style="border: 1px solid black; padding: 2px;">9</span>	<span style="border: 1px solid black; padding: 2px;">Y</span>
7	8	9	10	11	12		17	43	44		47	48

## CAUSE DESCRIPTION

<span style="border: 1px solid black; padding: 2px;">08</span>	Nonequilibrium xenon condition existed during the rod withdrawal	80
<span style="border: 1px solid black; padding: 2px;">09</span>	periods. Reactor Engineering will investigate optimum startup	80
<span style="border: 1px solid black; padding: 2px;">10</span>	conditions to be available in future.	80

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION					
<span style="border: 1px solid black; padding: 2px;">11</span>	<span style="border: 1px solid black; padding: 2px;">F</span>	<span style="border: 1px solid black; padding: 2px;">0</span> <span style="border: 1px solid black; padding: 2px;">4</span> <span style="border: 1px solid black; padding: 2px;">8</span>	<span style="border: 1px solid black; padding: 2px;">B</span>	Normal Shift Surveillance					
7	8	9	10	12	13	44	45	46	
FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE						
<span style="border: 1px solid black; padding: 2px;">12</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>	NA						
7	8	9	10	11	44	45			80

## PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION						
<span style="border: 1px solid black; padding: 2px;">13</span>	<span style="border: 1px solid black; padding: 2px;">0</span> <span style="border: 1px solid black; padding: 2px;">0</span> <span style="border: 1px solid black; padding: 2px;">0</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>						
7	8	9	11	12	13		NA	80

## PERSONNEL INJURIES

NUMBER	DESCRIPTION						
<span style="border: 1px solid black; padding: 2px;">14</span>	<span style="border: 1px solid black; padding: 2px;">0</span> <span style="border: 1px solid black; padding: 2px;">0</span> <span style="border: 1px solid black; padding: 2px;">0</span>						
7	8	9	11	12		NA	80

## OFFSITE CONSEQUENCES

<span style="border: 1px solid black; padding: 2px;">15</span>	NA	80
----------------------------------------------------------------	----	----

## LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION					
<span style="border: 1px solid black; padding: 2px;">16</span>	<span style="border: 1px solid black; padding: 2px;">Z</span>					
7	8	9	10		NA	80

## PUBLICITY

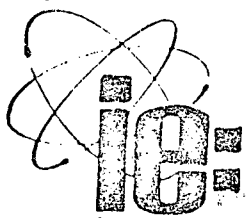
<span style="border: 1px solid black; padding: 2px;">17</span>	NA	80
----------------------------------------------------------------	----	----

## ADDITIONAL FACTORS

<span style="border: 1px solid black; padding: 2px;">18</span>		80
<span style="border: 1px solid black; padding: 2px;">19</span>		80

NAME: Ellery L. Hammond

PHONE: 319-851-5611



Iowa Electric light and power company

DUANE ARNOLD ENERGY CENTER  
PALO, IOWA

June 9, 1976

# MCPR Violation Report for 6/1/76

File: J-40b

On May 31, 1976 and June 1, 1976 control rod withdrawals per IPOI Appendix I, Table II.G.3-1 for the A-2 rod sequence were being made to maneuver to a higher load line prior to increasing power using recirc flow. A summary of the operation early on June 1 is contained in Table 1.

At 0431 hours and 0630 hours CPRRAT for 8 x 8 fuel assemblies exceeded 1.00 indicating a violation of the Tech Spec Minimum Critical Power Ratio limit. In both instances action was taken within 15 minutes to return MCPR within limits. Both violations were in limits again within 1 hour of the first indications of the violations.

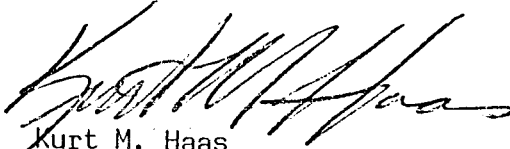
While a non-equilibrium Xenon condition (low Xenon inventory) was tending to decrease the value of CPR during the period of the violations, two calculational problems existed at the same time.

Due to an oversight, General Electric did not communicate to Iowa Electric the fact that the fuel loading for cycle 2 is mirror symmetric. From startup in the A-2 sequence until 1059 hours on June 1, the process computer core limits calculations were using rotational symmetry. In this instance the rotational symmetry mode produced overly conservative limits. The result of switching from rotational to mirror symmetry is shown in Table 1. The difference between the two modes is of the same order of magnitude as the amount by which the MCPR limit was exceeded.

An examination of the number of base critical codes that accompanied the calculation of the violations shows that at 0431 hours 7 base crit's were present. At 0630 hours 15 base crit's were present. Numerous examples of base critical codes affecting the core limits calculation in a conservative direction can be cited to argue that the MCPR violations were actually the product of overly conservative calculations due to the existence of several base critical codes at the time of the calculation.

Either of the above calculation problems could have led to erroneously low MCPR's resulting in apparent violations. It is very probable that the combination of the two problems caused erroneously low MCPR's. Unfortunately, the requirement of Tech Spec for prompt action to bring MCPR within limits precludes the use of detailed study and recalculation to determine the real source of the problem at the time it occurs. Investigation of the problem at a later time usually provides inconclusive results.

Thus, in the absence of concrete proof to the contrary, the MCPR violations of June 1 were due to a low Xenon concentration condition. A nearer equilibrium Xenon condition would have supported the rod pattern without a MCPR violation. To prevent a recurrence of this problem, studies should be performed to determine optimum startup strategy.

  
Kurt M. Haas  
Reactor Engineer  
Duane Arnold Energy Center

REVIEWED BY: Elaine L. Hammond DATE: 6/10/76  
Chairman Operations Committee

KMH/mg

cc: Mr. E. Hammond  
Mr. B. York  
Mr. W. NoDean

TABLE 1

TIME	8x8 CPRRAT	MCPR	REQUIRED MCPR	POWER MWT	FLOW Mlb/hr	BASE CRIT's	SYMMETRY MODE	CONTROL ROD MOVEMENTS
0130	0.969	1.829	1.773	760	18.43	1	Rotational	
0330	0.995	1.772	1.763	802	19.24	6	"	Group 42 @ 12
0431	1.001	1.759	1.761	807	19.39	7	"	
0515	0.984	1.793	1.764	784	19.11	3	"	Group 42 @ 10
0531	0.976	1.805	1.763	783	19.24	3	"	
0555	0.972	1.776	1.727	839	21.96	6	"	
0630	1.017	1.689	1.717	894	22.69	15	"	Group 42 @ 12
0654	0.987	1.739	1.717	868	22.74	10	"	Group 42 @ 10
0730	0.987	1.733	1.710	878	23.25	11	"	
0830	0.995	1.708	1.699	898	24.07	10	"	
0906	0.967	1.757	1.699	874	24.07	5	"	Group 42 @ 8
0932	0.962	1.766	1.699	871	24.06	4	"	
1030	0.964	1.763	1.700	867	24.00	8	"	
1059	0.951	1.788	1.699	866	24.06	5	Mirror	

*D. Lanham*

# IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER  
P. O. Box 351  
Cedar Rapids, Iowa 52406

June 10, 1976  
DAEC -76 -189

**50-331**

Mr. James G. Keppler, Director  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission - Region III  
799 Roosevelt Road  
Glen Ellyn, Illinois 60137

Subject: Licensee Event Report No. 76-37  
(30 day)

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center and Regulatory Guide 10.1, please find attached a copy of the subject Licensee Event Report. (Total of 3 copies transmitted)

Very truly yours,

*G. G. Hunt*

G. G. Hunt  
Chief Engineer  
Duane Arnold Energy Center

Docket 50-331

attachment

GGH/DLW/mg

cc: Director, Office of Inspection and Enforcement (30)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Director, Management Information and Program Control (3)  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

# LICENSEE EVENT REPORT

CONTROL BLOCK: 1         6

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME														LICENSE NUMBER														LICENSE TYPE						EVENT TYPE			
01	I	A	D	A	C	1	0	0	-	0	0	0	0	-	0	0	4	1	1	1	1	0	3														
7	8	9				14	15									25	26					30	31	32													

CATEGORY				REPORT TYPE		REPORT SOURCE		DOCKET NUMBER								EVENT DATE						REPORT DATE					
01	CONT			L	L			0	5	0	-	0	3	3	1	0	6	0	1	7	6	0	6	1	0	7	6
7	8		57	58	59	60	61								68	69					74	75					80

**EVENT DESCRIPTION**

02	During control rod withdrawals, two MCPR violations occurred. First	80
03	one, MCPR reached 1.759 (limit 1.761); second one, MCPR reached 1.689	80
04	(limit 1.717). Group 42 rods inserted one notch in both cases to re-	80
05	turn MCPR within limits. (RO 76-37).	80
06		80

SYSTEM CODE		CAUSE CODE		COMPONENT CODE						PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER						VIOLATION	
07	R	C	D	Z	Z	Z	Z	Z	Z	Z	Z	9	9	9	Y				
7	8	9	10	11	12				17	43		44			47	48			

**CAUSE DESCRIPTION**

08	Nonequilibrium xenon condition existed during the rod withdrawal	80
09	periods, Reactor Engineering will investigate optimum startup	80
10	conditions to be available in future.	80

FACILITY STATUS		% POWER		OTHER STATUS				METHOD OF DISCOVERY		DISCOVERY DESCRIPTION					
11	F	0	4	8	NA				B	Normal Shift Surveillance					
7	8	9	10	12	13			44	45	46				80	

FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE					
12	Z	Z	NA				NA						
7	8	9	10	11			44	45					80

**PERSONNEL EXPOSURES**

NUMBER			TYPE		DESCRIPTION							
13	0	0	0	Z	NA							
7	8	9	11	12	13							80

**PERSONNEL INJURIES**

NUMBER			DESCRIPTION									
14	0	0	0	NA								
7	8	9	11	12								80

**OFFSITE CONSEQUENCES**

15		80
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**LOSS OR DAMAGE TO FACILITY**

TYPE		DESCRIPTION										
16	Z	NA										
7	8	9	10									80

**PUBLICITY**

17		80
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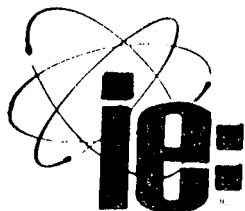
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NAME: Ellery L. Hammond

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Kurt M. Haas  
Reactor Engineer  
Duane Arnold Energy Center

REVIEWED BY: Ellery L. Hammond DATE: 6/10/76  
Chairman Operations Committee

KMH/mg

cc: Mr. E. Hammond  
Mr. B. York  
Mr. W. Nodean

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1030	0.964	1.763	1.700	867	24.00	8	"	
1059	0.951	1.788	1.699	866	24.06	5	Mirror	