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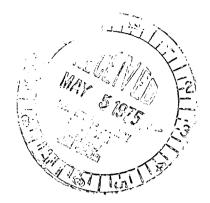
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General Office

CEDAR RAPIDS, IOWA

DUANE ARNOLD ENERGY CENTER
PALO, IOWA
APRIL 28, 1975
DAEC - 180 - 75



Mr. James G. Keppler, Director Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission - Region III 799 Roosevelt Road Glen Ellyn, Illinois 60137

SUBJECT: Abnormal Occurrence No. A. O. 50-331/75-20

FILE: A-110 A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center, please find enclosed a written report on the subject abnormal occurrence.

Mr. C. Feierabend, of your office, was notified of the abnormal occurrence on April 18, 1975.

Very truly yours,

G. G. Hunt Chief Engineer

Duane Arnold Energy Center

DLW/GGH/mg Enclosure

cc: B. C. Rusche

C. W. Sandford

J. A. Wallace

E. L. Hammond

H. W. Rehrauer - Chairman, Safety Committee

J. R. Newman



General Office

CEDAR RAPIDS, IOWA

Subject:

Abnormal Occurrence

Report Number:

A. 0. 50-331/75-20

Report Date:

April 28, 1975

Occurrence Date:

April 18, 1975

Facility:

Duane Arnold Energy Center, Unit #1, Palo, Iowa

Identification of Occurrence

Inoperable scram discharge volume level switch, reportable in accordance with Appendix A to Operating License DPR-49, Specifications 1.0.4.b and Table 3.1-1.

Description of Occurrence

During the performance of Surveillance Test Procedure No. 41A006 - Discharge Volume High Water Level, Level Switch 1861B did not operate at the required setpoint. An immediate investigation by maintenance personnel determined that the float in the level switch had stuck in the untripped condition.

Designation of Apparent Cause of Occcurrence

The cause of the occurrence is unknown. An inspection of the LS 1861B indicated no abnormal condition which would cause the float to stick.

It should be noted that another scram discharge volume level switch (LS 1861D) stuck in the tripped condition and would not reset following a reactor scram on April 19, 1975. Since a replacement level switch was readily available, the float reservoir for LS 1861D was dissassembled and inspected. The inspection indicated the probable cause was a foreign object in the float reservoir. A ½" diameter x ¼" thick piece of plastic was found in the reservoir. Also, the float stem was observed to be bent at an angle of approximately 8°. It is postulated that the piece of plastic may have wedged between the float and the reservoir wall. The bent float stem probably was the result of a manufacturing deficiency.

Analysis of Occurrence

LS 1861B provides a signal to the reactor protection system (trip Channel B) in the event of high level in the scram discharge volume. Since a redundant level switch in the same reactor protection system trip channel was found to

be operable and within calibration during the surveillance test on April 28, 1975, the occurrence did not present an unsafer plant condition. In addition, another level switch in the scram discharge volume (LS 1861F) would have alarmed in the control room and alerted operating personnel to an increasing level condition in the scram discharge volume.

Corrective Action

The mechanical portion of LS 1861B, excluding the float reservoir, was disassembled, cleaned, lubricated and reassembled. The switch was also recalibrated and functionally tested.

The level switch vendor will conduct an investigation into the cause of the switch malfunction.

The frequency for performance of Surveillance Test No. 41A006 has been increased to bi-weekly pending completion of the investigation. A supplemental report summarizing the results of the vendor's investigation will be submitted.

Failure Data

There have been no previous malfunctions of this particular model level switch at this facility.

Equipment Identification

Description: Level Switch

Manufacturer: Magnetrol

Model No. 5.0-751-0.75-1X-MPG-M13HV

Conclusion

This report was reviewed and approved by the DAEC Operations Committee on April 28, 1975. The Committee concluded the occurrence did not present a hazard to the health and safety of the public.

G. G. Hunt

Chief Engineer

Duane Arnold Energy Center

Theut

DLW/GGH/mg

General Office

CEDAR RAPIDS IOWA DUANE ARNOLD ENERGY CENTER PALO, IOWA APRIL 18, 1975 DAEC-75-167

Mr. James G. Keppler, Director Office of Inspection and Enforcement-Region III U. S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

> SUBJECT: Abnormal Occurrence No. A.O. 50-331/75-20 FILE: A-110 A-118a

Dear Mr. Keppler:

In accordance with DAEC Technical Specifications, Specification 6.11.2.A.1, this letter is a notification of an abnormal occurrence at the Duane Arnold Energy Center on April 18, 1975. This letter, telecopied to your office, is intended to satisfy the requirements for notification by telegram within 24 hours of an abnormal occurrence.

Identification of Occurrence

Inoperable scram discharge volume level switch, reportable in accordance with Appendix A to Operating License DPR-49, Specifications 1.0.4.b and Table 3.1-1.

Description of Occurrence

During the performance of Surveillance Test Procedure No. 41A006 - Discharge Volume High Water Level, Level Switch 1861 B did not trip at the required setpoint. An immediate investigation determined that the float in the level switch was sticking.

Reactor Protection System Channel B was placed in the tripped condition pending completion of an investigation into the cause of the malfunction and completion of appropriate corrective action.

Investigation is continuing and a formal written report will be submitted to the Office of Inspection and Enforcement within 10 days of the subject # HORD abnormal occurrence.

G. G. Hunt Chief Engineer

Duane Arnold Energy Center

DLW: GGH: bh

CC: ✓B. C. Rusche

C. W. Sandford

J. A. Wallace

L. D. Root

B. R. York

D. L. Wilson

R. R. Rinderman

G. A. Cook

H. W. Rehrauer-Chairman, Safety Committee

E. L. Hammond

B. L. Hopkins

General Office

CEDAR RAPIDS, IOWA DUANE ARNOLD ENERGY CENTER PALO, IOWA APRIL 18, 1975 DAEC-75-169

Mr. James G. Keppler, Director Office of Inspection and Enforcement Region III U. S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

> SUBJECT: Abnormal Occurrence No. A.O. 50-331/75-21

FILE: A-110 A-118a

Dear Mr. Keppler:

In accordance with DAEC Technical Specifications, Specification 6.11.2.A.1, this letter is a notification of an abnormal occurrence at the Duane Arnold Energy Center on April 18, 1975. This letter, telecopied to your office, is intended to satisfy the requirements for notificaiton by telegram within 24 hours of an abnormal occurrence.

Identification of Occurrence

HPCI Subsystem Local Leak Rate test line fracture, reportable in accordance with Appendix A to Operating License DPR-49, Specification, 1.0.4.e.

Description of Occurrence

During the performance of a HPCI Subsystem valve lineup verification, a 3/4" local leak rate test line was observed to have fractured at the point where the line connects to the 16" HPCI turbine exhaust line just upstream of valve V-22-17.

An investigation into the cause of the pipe fracture is now in progress. Valve V-22-17 has been closed to ensure primary containment pending completion of corrective action.

A formal written report will be submitted to the Office of Inspection and Enforcement within 10 days of the subject abnormal occurrence.

G. G. Hunt Chief Engineer

Duane Arnold Energy Center

DLW/GGH/mg

cc: B. C. Rusche

C. W. Sandford

L. D. Root

G. G. Hunt

H. W. Rehrauer-Chr. SC

B. R. York

R. R. Rinderman

B. L. Hopkins

J. A. Wallace

D. L. Wilson

G. A. Cook