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FILE: INCIDENT REPORT FILE

FROM: Iowa Elec. Light & Pwr. Co. Palo, Iowa G.G.Hunt		DATE OF DOC 7-31-75	DATE REC'D 8-6-75	LTR XX	TWX	RPT	OTHER
TO: Mr. J. G. Keppler		ORIG 1 signed	CC	OTHER	SENT AEC PDR <u>XX</u>		SENT LOCAL PDR <u>XX</u>
CLASS	UNCLASS XXX	PROP INFO	INPUT	NO CYS REC'D 1	DOCKET NO: 50-331/XXXXX		

DESCRIPTION: Ltr trans the following:

ENCLOSURES: Abnormal Occurrence AO-50-331/
75-41 on 7-24-75 re failure of Pressure
Switch PS-4552.....

(1 Orig y enc'l rec'd)

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ACKNOWLEDGED

PLANT NAME: Duane Arnold Plant

FOR ACTION/INFORMATION

DHL 8-8-75

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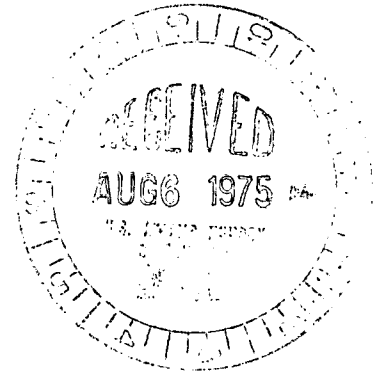
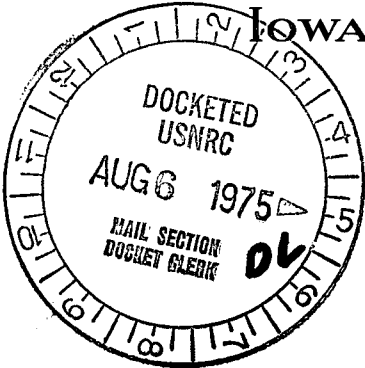
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IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office

CEDAR RAPIDS, IOWA
DUANE ARNOLD ENERGY CENTER
PALO, IOWA
JULY 31, 1975
DAEC -75 -304



Mr. James G. Keppler, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission - Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Subject: Abnormal Occurrence No. AO 50-331/75-41

File: A-110, A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center, please find enclosed a written report on the subject abnormal occurrence.

Mr. C. D. Feierabend, of your office, was notified of the abnormal occurrence by telephone on July 24, 1975.

Very truly yours,

G. G. Hunt
Chief Engineer
Duane Arnold Energy Center

OCS/GGH/mg

cc: B. C. Rusche
C. W. Sandford
J. A. Wallace
L. Liu
H. W. Rehrauer - Chairman, Safety Committee
J. R. Newman
E. L. Hammond

8380

AUG 4 1975

IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office

CEDAR RAPIDS, IOWA

Subject: Abnormal Occurrence Date: 7-31-75
Report Number: AO 50-331/75-41
Report Date: July 29, 1975
Occurrence Date: July 24, 1975
Facility: Duane Arnold Energy Center, Unit No. 1, Palo, Iowa

Identification of Occurrence

Reactor High Pressure trip level setting reportable per Appendix A, Operating License DPR-49, Specification 1.0.4.b and Table 3.1-1.

Description of Occurrence

During the performance of Surveillance Test Procedure STP 41A001, "Reactor High Pressure (RPS) Instrument Functional Test and Calibration", Pressure Switch (PS) 4552 was found at 1080 psig. Table 3.1-1 lists the correct trip level setting as 1035 psig. To this setting (1035) is added 26 psi for water leg correction and an instrument tolerance of plus or minus 12 psig. The correct trip level setting is therefore 1061 ± 12 psig.

Designation of Apparent Cause of Occurrence

The apparent cause of the occurrence was instrument drift.

Analysis of Occurrence

It has been determined that the occurrence did not have a deleterious affect on plant safety. Reactor protection from high pressure was still provided in that one reactor pressure switch in Trip System B and both reactor pressure switches in Trip System A were fully operable.

Corrective Action

PS 4552 was immediately recalibrated. The trending program of previous test results on this pressure switch did not show a trend to an out-of-calibration condition.

Conclusion

This report was reviewed and approved by the DAEC Operations Committee on July 29, 1975. The Committee concluded that the occurrence did not present a hazard to the health and safety of the public.



G. G. Hunt
Chief Engineer
Duane Arnold Energy Center