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Iowa Elec. Light & Power Cedar Rapids, Iowa E.L. Hammond	11-28-75	12-5-75	XXX			

TO:	ORIG	CC	OTHER	SENT AEC PDR	SENT LOCAL PDR
James G. Keppler	1 Signed	0		XXX	XXX

CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:
	XXX			1	50-331

DESCRIPTION:
Letter trans the following.....

ENCLOSURES:
Abnormal Occurrence # 75-63, on 11-19-75, Concerning Reactor Low Pressure switches being out of specification.....

(1 Copy Received)

PLANT NAME: Duane Arnold

FOR ACTION/INFORMATION SAB 12-8-75

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IOWA ELECTRIC LIGHT AND POWER COMPANY

Regulatory File Cy

DUANE ARNOLD ENERGY CENTER
P. O. Box 351
Cedar Rapids, Iowa 52406

DAEC - 75 - 437

November 28, 1975

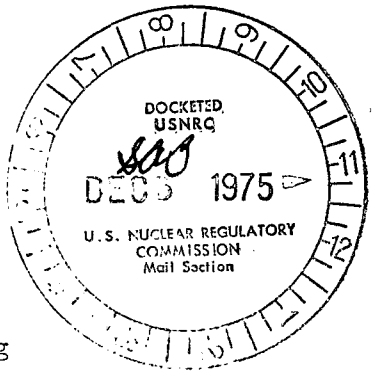
Mr. James G. Keppler, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission-Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Subject: Abnormal Occurrence No. AO 50-331/75-63

File: A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center, please find enclosed a written report on the subject abnormal occurrence.



Very truly yours,

E. L. Hammond
Assistant Chief Engineer
Duane Arnold Energy Center

DLW/ELH/mg

cc: B. C. Rusche
Duane Arnold
J. A. Wallace
L. Liu
H. W. Rehrauer - Chairman, Safety Committee
J. R. Newman
G. G. Hunt

13642

DEC 1

IOWA ELECTRIC LIGHT AND POWER COMPANY

DUANE ARNOLD ENERGY CENTER
P. O. Box 351
Cedar Rapids, Iowa 52406

Subject: Abnormal Occurrence

Report Number: AO 50-331/75-63

Report Date: November 28, 1975

Occurrence Date: November 19, 1975

Facility: Duane Arnold Energy Center, Palo, Iowa

Identification of Occurrence

Out-of-specification Reactor Low Pressure (RHR Shutdown Cooling Isolation) pressure switches, reportable in accordance with Appendix A to Operating License DPR-49, Specifications 1.0.4.a and 3.2.A.

Conditions Prior to Occurrence

Reactor at rated pressure and temperature.

1295 MWth 438 MWe

Description of Occurrence

While performing Surveillance Test Procedure No. 42A002 - Reactor Low Pressure (RHR Shutdown Cooling), maintenance personnel incorrectly calibrated pressure switches 4637, 4638A and 4638B. These switches provide an interlock function to isolate and prevent operation of the RHR system in the shutdown cooling mode when reactor pressure is > 135 psig. In accordance with the surveillance test procedure, the switches should have been calibrated using increasing pressure. However, the maintenance technicians incorrectly calibrated the switches using decreasing pressure, and as a result, the trip point of the switches for increasing reactor pressure was approximately 160 psig and was above the 135 psig setpoint required by the Technical Specifications. The switches were in an out-of-specification condition for approximately three hours.

Designation of Apparent Cause of Occurrence

The cause of the occurrence was personnel error. Maintenance personnel performing the surveillance test did not comply with the requirements of the approved test procedure.

Analysis of Occurrence

The occurrence did not present an unsafe plant condition. Even though the setpoint of the switches was out-of-specification for a short time, reactor pressure was well above this point and the RHR Shutdown Cooling System remained isolated as required.

Corrective Action

Pressure switches 4637, 4638A and 4638B were recalibrated when the error in calibration was discovered.

Maintenance personnel involved have been reinstructed as to the requirements for strictly adhering to the requirements of surveillance test procedures.

Conclusion

This report was reviewed and approved by the DAEC Operations Committee on November 28, 1975. The Committee concluded that the occurrence did not present a hazard to the health and safety of the public.

E. L. Hammond

E. L. Hammond
Assistant Chief Engineer
DAEC

DLW/ELH/mg