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TO:	ORIG	CC	OTHER	SENT AEC PDR	SENT LOCAL PDR
James G. Keppler	1Signed			XXXX	XXXX

CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:
				40	331

DESCRIPTION:

Ltr. teans the following....

ENCLOSURES:

Abnorm. Occurr. # 75-14, on 3-23-75 concerning Unidentified coolant leakage into the primary containment....

ACKNOWLEDGED

(40cy. Encl. Rec'd)

DO NOT REMOVE

PLANT NAME: Duane Arnold

FOR ACTION/INFORMATION VCR 4-11-75

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IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office

CEDAR RAPIDS, IOWA

DUANE ARNOLD ENERGY CENTER

PALO, IOWA

MARCH 31, 1975

DAEC-75-131

Mr. James G. Keppler, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission-Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

SUBJECT: Abnormal Occurrence No. A.O. 50-331/75-14
FILE: A-110
A-118a

Dear Mr. Keppler:

In accordance with Appendix A to Operating License DPR-49, Technical Specifications and Bases for Duane Arnold Energy Center, please find enclosed a written report on the subject abnormal occurrence.

Very truly yours,

G. G. Hunt
Chief Engineer
Duane Arnold Energy Center

DLW:GGH:bh

CC:E. G. Case
C. W. Sandford
J. A. Wallace
E. L. Hammond
D. L. Wilson
H. W. Rehrauer-Chairman, Safety Committee

3881

IOWA ELECTRIC LIGHT AND POWER COMPANY

General Office

CEDAR RAPIDS, IOWA

Subject:	Abnormal Occurrence
Report Number:	A.O. 50-331/75-14
Report Date:	March 31, 1975
Occurrence Date:	March 23, 1975
Facility:	Duane Arnold Energy Center, Unit #1, Palo, Iowa

Identification of Occurrence

Unidentified coolant leakage into the primary containment, reportable in accordance with Appendix A to Operating License DPR-49, Specifications 1.0.4.b and 3.6.C.1.

Conditions Prior to Occurrence

Reactor at steady-state power conditions, 430 MWt and 119 MWe.

Unidentified drywell coolant leakage rate - 0.6 gpm.
Identified leakage rate - 2.0 gpm.

Description of Occurrence

At approximately 0245 hours, the following alarms were received in the Control Room:

Hi Drywell Equipment Drain Sump Temperature
Drywell Equipment Drain Sump - Hi Leak

An immediate investigation by operating personnel determined that the drywell equipment drain and drywell floor drain integrators were indicating 21.3 gpm and 10.4 gpm respectively. A later analysis of the rate of accumulation of water in the radwaste processing facilities indicated that the actual equipment drain and floor drain leakage rates were 14.6 gpm and 4.0 gpm, respectively. The high readings on the sump flow integrators apparently were caused by the high temperature water flashing to steam across the flow elements.

Preparations for a reactor shutdown and subsequent drywell entry were commenced at 0315 hours.

The main turbine-generator was removed from the grid at 0700 hours.

Designation of Apparent Cause of Occurrence

The cause of the occurrence was an internal packing leak on the "B" Recirculation Loop discharge valve. (MOV 4628) Leakage from the packing is piped directly to the drywell equipment drain sump. The increased leakage to the drywell floor drain sump was caused by overflow from the equipment drain sump.

Analysis of Occurrence

All of the leakage from the packing on MOV 4628 was handled by the drywell equipment and floor drain sumps and processed as liquid radwaste, therefore, the occurrence did not present a hazard to the health and safety of the public.

Corrective Action

MOV 4628 was backseated and the drain line from the internal packing to the drywell equipment drain sump was isolated. An inspection of the drywell verified that the outer packing on MOV 4628 was not leaking. Any future leakage from the MOV 4628 outer packing will be detected by the drywell floor drain sump and classified as unidentified leakage for determination of Technical Specification limits.

Conclusion

This report was reviewed and approved by the DAEC Operations Committee on March 31, 1975. The Committee concluded that the occurrence did not present a hazard to the health and safety of the public.

Very truly yours,

G. G. Hunt
Chief Engineer
Duane Arnold Energy Center

DLW:GGH:bh