

U.S. Nuclear Regulatory Commission
2011 HNP NRC
Site-Specific SRO Written Examination

Applicant Information

Name:

Date:

Facility/Unit: Harris Nuclear Plant

Region:

I ☐ II ☒ III ☐ IV ☐Reactor Type: W ☒ CE ☐ BW ☐ GE ☐

Start Time:

Finish Time:

Instructions

Use the answer sheets provided to document your answers. Staple this cover sheet on top of the answer sheets. To pass the examination you must achieve a final grade of at least 80.00 percent overall, with 70.00 percent or better on the SRO-only items if given in conjunction with the RO exam; SRO-only exams given alone require a final grade of 80.00 percent to pass. You have 8 hours to complete the combined examination, and 3 hours if you are only taking the SRO portion.

Applicant Certification

All work done on this examination is my own. I have neither given nor received aid.

Applicant's Signature

Results

RO/SRO-Only/Total Examination Values

75 / 25 / 100 Points

Applicant's Scores

____ / ____ / ____ Points

Applicant's Grade

____ / ____ / ____ Percent

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76. Given the following plant conditions:

- The plant is in Mode 6
- An assembly is in the Manipulator Crane
- A 140 gpm RCS leak occurs
- The crew has adjusted Charging flow
- Reactor cavity level has been stabilized at 23.2' above the Reactor Flange
- The crew has just entered AOP-020, Loss of RCS Inventory or Residual Heat Removal While Shutdown

Which ONE of the following (1) describes the applicable procedure to be used for this condition AND (2) the required action to respond to this condition?

(RCS temperature will remain below 200°F)

- A. (1) Remain in AOP-020;
(2) Initiate flow through the BIT
- B. (1) Remain in AOP-020;
(2) Control Charging flow through the normal charging line
- C. (1) Transition to AOP-031 Loss of Refueling Cavity Integrity;
(2) Initiate flow through the BIT
- D. (1) Transition to AOP-031 Loss of Refueling Cavity Integrity;
(2) Control Charging flow through the normal charging line

77. Given the following plant conditions:

- The plant is in Mode 4
- RCS makeup capability is limited to 80 gpm
- RHR Train 'B' has been placed in service
- RCS temperature is stable at 325°F
- RCS pressure is 320 psig and lowering
- Containment sump level is 20 inches and rising
- Pressurizer level is 28% and lowering

The OAC responded by taking manual control of FK-122.1 and has increased Charging flow to 90 gpm. Pressurizer level is now stable.

Which ONE of the following describes (1) the correct procedure to address the event in progress AND (2) the mitigative actions this procedure will direct?

- A. (1) AOP-016, Excessive RCS Leakage
(2) Stop all running RHR pumps and isolate RHR
- B. (1) AOP-016, Excessive RCS Leakage
(2) Establish BIT flow
- C. (1) AOP-020, Loss of RCS Inventory or Residual Heat Removal While Shutdown
(2) Stop all running RHR pumps and isolate RHR
- D. (1) AOP-020, Loss of RCS Inventory or Residual Heat Removal While Shutdown
(2) Establish BIT flow

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78. Given the following plant conditions:

- Reactor power is 7%
- RCS pressure is at 1900 psig and lowering
- 'A' SG Levels are: LI-474 = 27%, LI-475 = 23%, LI-476 = 23%, LI-473 = 27%
- An automatic Reactor trip has not occurred and attempts to manually trip the Reactor from both trip switches are unsuccessful
- Immediate actions of FRP-S.1 have just been completed

Which ONE of the following:

- (1) Describes the Reactor Trip annunciator (Reactor first out) in alarm?
 - (2) If a Safety Injection occurs now is PATH-1 GUIDE, Attachment 6, Safeguards Actuation Verification required to be performed at this time?
- A. (1) REACTOR TRIP STEAM GEN-A LOW-LOW LEVEL
- (2) Yes
- B. (1) REACTOR TRIP PRESSURIZER LOW PRESS
- (2) Yes
- C. (1) REACTOR TRIP STEAM GEN-A LOW-LOW LEVEL
- (2) No
- D. (1) REACTOR TRIP PRESSURIZER LOW PRESS
- (2) No

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79. Which ONE of the following satisfies the Tech Spec bases requirement for off-site power distribution with the plant in Mode 1?
- A. The requirement can be satisfied by any two separate off-site transmission lines that can power the SUTs.
 - B. The requirement can ONLY be satisfied by two off-site transmission lines that feed the SUTs directly (Cary Regency Park and Cape Fear North).
 - C. The requirement can ONLY be satisfied by the off-site transmission lines that do not feed the respective North or South switchyard bus through a jumper.
 - D. The requirement is satisfied as long as the switchyard alignment is such that power is available from the off-site transmission network to both SUTs regardless of the number of transmission lines available.

80. Given the following plant conditions:

- The plant is operating at 100% power
- ALB-015-4-5, Channel III UPS Trouble has just alarmed
- Feed flows to all SG's have not changed

Which ONE of the following completes the statements below?

The Instrument Bus III Inverter input has lost (1). IAW Tech Spec 3.8.3.1 the normal power supply alignment must be restored to the Inverter within (2) hours.

- A. (1) DC power ONLY
(2) 2
- B. (1) DC power ONLY
(2) 24
- C. (1) AC and DC power
(2) 2
- D. (1) AC and DC power
(2) 24

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81. Given the following plant conditions:

- The plant has experienced a Reactor Trip and Safety Injection
- The crew has implemented PATH-1
- A transition was made to EPP-013, LOCA Outside Containment
- The RAB radiological conditions are being evaluated by HP and currently HP will NOT allow any personnel entry

Which ONE of the following parameters will be used to determine if the break is isolated IAW EPP-013, AND the procedure transitioned to when the leak is isolated?

- A. RCS pressure increasing
PATH-1, entry point C
- B. Pressurizer level increasing
PATH-1, entry point C
- C. RCS pressure increasing
EPP-012, Loss Of Emergency Coolant Recirculation
- D. Pressurizer level increasing
EPP-012, Loss Of Emergency Coolant Recirculation

82. Given the following plant conditions:

- The plant is operating at 100% power
- Pressurizer Level Instrument, LT-461, is isolated due to a leak on the transmitter
- 'A' CSIP has been under clearance for electrical inspection
- Pressurizer Level Controller Selector switch is selected to 459/460

The OAC reports the following indications:

- EI-222, B CSIP motor amps, indicates 45 amps
- PI-121, Charging HDR Press, indicates 2800 psig
- Pressurizer level is lowering on Pressurizer Level Instrument LT-460

Which ONE of the following describes (1) the malfunction AND (2) the required action?

- A. (1) Pressurizer Level Instrument, LT-459, has failed high
(2) Place the unit in Mode 3 within 7 hours
- B. (1) Pressurizer Level Instrument, LT-459, has failed high
(2) Place Level Channel 459 in a tripped condition within 6 hours
- C. (1) The B CSIP has a shaft shear
(2) Place the unit in Mode 3 within 7 hours
- D. (1) The B CSIP has a shaft shear
(2) Restore one CSIP to an OPERABLE status within 72 hours

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83. Given the following plant conditions:

- The plant is operating at 100% power
- Degrading Condenser vacuum was observed
- CTMP-7-1, Cooling Tower 1 Level HI/LO is in alarm
- AOP-012, Partial Loss of Condenser Vacuum has been entered
- IAW AOP-012, The Reactor has been tripped, PATH-1 has been entered

Which ONE of the following describes (1) a condition/parameter that would require the operator to continue taking actions IAW AOP-012, AND (2) when will it be appropriate to take those actions?

- A. (1) Condenser Pit High Level alarm annunciates
(2) Continue performing the actions of AOP-012 only after EPP-004 is exited.
- B. (1) Condenser Pit High Level alarm annunciates
(2) Continue performing the actions of AOP-012 during the performance EPP-004 as time allows
- C. (1) Continuous running of both Industrial Waste Sump pumps is observed
(2) Continue performing the actions of AOP-012 only after EPP-004 is exited
- D. (1) Continuous running of both Industrial Waste Sump pumps is observed
(2) Continue performing the actions of AOP-012 during the performance EPP-004 as time allows

84. The crew is implementing FRP-J.2, Response to Containment Flooding

Which ONE of the following:

- (1) Describes the basis for obtaining Containment sump activity samples in FRP-J.2?
 - (2) Describes when a transition from FRP-J.2 to the procedure and step in effect is allowed?
- A. (1) To determine if ESW to Containment Fan Coolers should be isolated.
- (2) After completion of required actions in FRP-J.2 even if the associated Orange Path still exists.
- B. (1) To determine if ESW to Containment Fan Coolers should be isolated.
- (2) ONLY after the condition causing the associated Orange Path has been corrected.
- C. (1) To determine if sump water can be transferred to tanks outside containment.
- (2) ONLY after the condition causing the associated Orange Path has been corrected.
- D. (1) To determine if sump water can be transferred to tanks outside containment.
- (2) After completion of required actions in FRP-J.2 even if the associated Orange Path still exists.

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85. Initial Conditions:

- A LOCA has occurred
- The crew is performing actions of EPP-012, Loss of Emergency Coolant Recirculation, based on plant conditions upon transition from PATH-1
- RWST level is 2.9%

Current Conditions:

- RCS Integrity CSF Status Tree indicates ORANGE

Which ONE of the following describes the action and procedure usage required?

A. Stop all pumps taking suction from the RWST;

Go to FRP-P.1, Response to Imminent Pressurized Thermal Shock.

B. Align the RHR pump suction to Containment Recirc Sump;

Go to FRP-P.1, Response to Imminent Pressurized Thermal Shock.

C. Stop all pumps taking suction from the RWST;

Remain in EPP-012 because actions in EPP-012 are expected to cause an Orange condition on Integrity.

D. Align the RHR pump suction to Containment Recirc Sump;

Remain in EPP-012 because actions in EPP-012 are expected to cause an Orange condition on Integrity.

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86. Given the following plant conditions:

- The plant is in MODE 3
- RCS pressure is 2235 psig
- RCS Tavg is 557°F
- Chemistry reports the 'B' SI Accumulator boron concentration is 2392 ppm

Which ONE of the following completes the statements below?

(Each statement is to be evaluated independently)

The 'B' SI Accumulator boron concentration must be restored to within Tech Spec 3.5.1 limits within (1) hour(s).

The lineup for simultaneous fill and drain of 'B' SI Accumulator per OP-110, Safety Injection, will make (2) inoperable.

- A. (1) 1
(2) ONLY the RWST
- B. (1) 1
(2) BOTH the RWST and 'B' SI Accumulator
- C. (1) 72
(2) ONLY the RWST
- D. (1) 72
(2) BOTH the RWST and 'B' SI Accumulator

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87. Given the following plant conditions:

- The plant is operating at 100% power
- OST-1010A, A Train Containment Cooling System Operability Test is in progress
- Containment Fan Cooling Unit AH-2 A-SA, is in operation in low speed
- Containment Fan Cooling Unit AH-2 A-SA is selected as the lead fan
- ALB-027-7-2, Containment Fan Coolers AH-2 LOW FLOW O/L has just alarmed

The BOP operator reports the AH-2 A-SA control switch white light is illuminated.

Which ONE of the following:

- 1) Describes the required Tech Spec 3.6.2.3 action?
 - 2) Describes the action that will restore Train SA Containment Fan Cooler operability?
- A. (1) Restore the inoperable train of fan coolers to operable within 7 days.
(2) Select AH-2 B-SA as the lead fan.
- B. (1) Restore the inoperable train of fan coolers to operable within 7 days.
(2) Select AH-3 A-SA as the lead fan.
- C. (1) Restore the inoperable train of fan coolers to operable within 72 hours.
(2) Select AH-3 A-SA as the lead fan.
- D. (1) Restore the inoperable train of fan coolers to operable within 72 hours.
(2) Select AH-2 B-SA as the lead fan.

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88. Given the following initial plant conditions:

- 'B' Main Steam Line radiation monitor is in HIGH alarm
- The operating crew initiated a Reactor trip and Safety Injection
- Both MDAFW pumps are unavailable due to common cause motor problems

- After the Reactor Trip, one 'B' SG safety valve stuck open
- MSIV's will not close

Which ONE of the following describes:

- (1) The required operation of 1MS-70, MAIN STEAM B TO AUX FW TURBINE
- (2) Which procedure will be used to close 1MS-70

- A. (1) maintained open
 - (2) EOP-EPP-014, Faulted Steam Generator Isolation
- B. (1) maintained closed
 - (2) EOP-EPP-014, Faulted Steam Generator Isolation
- C. (1) maintained open
 - (2) PATH-2
- D. (1) maintained closed
 - (2) PATH-2

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89. The plant is operating at 100% power. The following MCB indications exist:

	Time	<u>1000</u>	<u>1003</u>	<u>1005</u>
<u>A ESW Header</u>				
PI-9101A SA, A Disch Press		87 psig	84 psig	78 psig
FI-9101A1 SA, A Hdr Flow		15650 gpm	18650 gpm	20650 gpm
<u>B ESW Header</u>				
PI-9101B SB, B Disch Press		89 psig	85 psig	95 psig
FI-9101B1 SB, B Hdr Flow		14450 gpm	11760 gpm	13380 gpm
<u>NSW Header</u>				
PI-9302.1, Disch Hdr Press		83 psig	79 psig	110 psig
FI-9304, WPB SW Flow		9850 gpm	8210 gpm	11820 gpm
FI-9276, Cnmt Coil Units SW Return		1800 gpm	1505 gpm	2875 gpm

AOP-022, Loss of Service Water has been entered. At 1004 the immediate actions and isolation of both the ESW and NSW headers have been completed.

Which ONE of the following completes the statements below?

The Service Water line rupture is located in the (1) ESW Header and Tech Spec 3.7.4 requires at least two loops are restored to operable status within (2) hours or be in at least Hot Standby within the next 6 hours.

- A. (1) 'A'
- (2) 24
- B. (1) 'B'
- (2) 24
- C. (1) 'A'
- (2) 72
- D. (1) 'B'
- (2) 72

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90. Given the following plant conditions:

- The plant is operating at 100% power
- 1ED-95, Containment Sump Pump Discharge valve is OPEN
- Breaker PM is being performed with the breaker OPEN

Which ONE of the following describes (1) the Tech Spec LCO that is not met AND (2) the applicable action statement?

- A. (1) TS 3.3.2 for Phase 'A' isolation is not met
(2) restore isolation capability within 4 hours.
- B. (1) TS 3.3.2 for Phase 'A' isolation is not met
(2) restore isolation capability within 6 hours.
- C. (1) TS 3.6.3 for Containment Isolation Valves is not met
(2) restore the valve to operable or isolate the penetration within 4 hours.
- D. (1) TS 3.6.3 for Containment Isolation Valves is not met
(2) restore the valve to operable or isolate the penetration within 6 hours.

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91. Given the following plant conditions:

- Following a Main Turbine runback from 100% power, the crew is stabilizing the plant IAW AOP-015, Secondary Load Rejection
- ALB-13-8-2, Bank Low Insertion Limit and ALB-13-8-3, Bank Low-Low Insertion Limit alarms remain locked in
- Control Bank 'D' Rods are at 93 steps
- No rod motion is possible due to a Rod Control Logic Cabinet Urgent Alarm

Which ONE of the following describes the MAXIMUM permitted Reactor Power and the time to complete the power reduction IAW Tech Specs?

(Reference Provided)

- A. Reduce Reactor Power to 50% in 2 hours
- B. Reduce Reactor Power to 50% in 4 hours
- C. Reduce Reactor Power to 44% in 2 hours
- D. Reduce Reactor Power to 44% in 4 hours

92. Given the following plant conditions:

- The plant is operating at 100% power Middle-Of-Life conditions
- A divergent xenon oscillation is in progress
- Axial Flux Differential (AFD) is NOT stable

Which ONE of the following identifies (1) the condition that will place the unit closest to the AFD limit AND (2) the Tech Spec basis for the AFD limit?

- A. (1) allowing AFD to trend to positive 6%
(2) assures that the radial power distribution satisfies the design values
- B. (1) allowing AFD to trend to positive 6%
(2) assures that the FQ(Z) limit in the COLR is NOT exceeded
- C. (1) allowing AFD to trend to negative 6%
(2) assures that the radial power distribution satisfies the design values
- D. (1) allowing AFD to trend to negative 6%
(2) assures that the FQ(Z) limit in the COLR is NOT exceeded

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93. Given the following initial plant conditions:

- Core offload activities are in progress
- Both Source Range Nuclear Instruments are OPERABLE
- N32 is selected for audible indication

- Source Range Nuclear Instrument N31 subsequently fails

Which ONE of the following completes the statement below?

Core offload activities may continue...

(Reference provided)

- A. when NFMS monitor N60 is used in place of N31.
- B. when NFMS monitor N61 is used in place of N31.
- C. with either NFMS monitor (N60 or N61) used in place of N31.
- D. with continuous visual monitoring of the operable Source Range N32.

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94. Initial plant conditions:

- Refueling activities are in progress.
- Fuel movement has stopped due to a problem with the Manipulator crane.

The Manipulator crane operator desires to operate TS-5, TROLLEY INTERLOCK BYPASS switch. IAW FHP-020, which ONE of the following describes the permission AND concurrence required to bypass the trolley interlock?

- A. The CRS must approve with the concurrence of the SM.
- B. The SRO-Fuel Handling must approve with the concurrence of the CRS.
- C. The SRO-Fuel Handling must approve with the concurrence of the SM.
- D. The SM must approve with the concurrence of the Maintenance Outage Manager of Refueling.

95. Given the following plant conditions:

- The plant is operating at 100% power.
- The low flow input from AH-1 A-SB to annunciator ALB-29/3-2, CONTAINMENT FAN COOLERS AH-1 LOW FLOW - O/L, is failed causing the annunciator to be lit when conditions are not valid.
- Repair of the failed low flow input is not expected to occur for at least one month.
- To enable this annunciator to function for AH-1 B-SB low flow and the overload condition on either fan, leads will be lifted to disable the annunciator input from the AH-1 A-SB low flow output.

IAW OPS-NGGC-1000, Fleet Conduct of Operations, the annunciator should be flagged (1) .

IAW OPS-NGGC-1308, Plant Status Control, (2) tags should be placed on the lifted leads.

- A. (1) blue
(2) Caution
- B. (1) blue
(2) Off Normal
- C. (1) yellow
(2) Caution
- D. (1) yellow
(2) Off Normal

96. Which ONE of the following contains the correct values to complete the statement below?

IAW AP-617, Reportability Determination and Notification, notification for exceeding an instantaneous thermal power level equivalent to ____ (1) ____ requires NRC notification within ____ (2) ____.

- A. (1) 103%
(2) 4 hours
- B. (1) 103%
(2) 24 hours
- C. (1) 102%
(2) 4 hours
- D. (1) 102%
(2) 24 hours

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97. A Source Check is being performed on the Plant Vent Stack Wide Range Gas Monitor, (RM-3509-1-SA).

The activity measured by the channel is lower than required when the source is exposed.

Which ONE of the following describes the expected indication and the correct Tech Spec action for this condition?

- A. (1) The Check Source (C/S) button on the RM-23 module flashes
(2) Initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours.
- B. (1) The Check Source (C/S) button on the RM-23 module flashes
(2) Restore the PVS WRGM to operable status within 72 hours or be in hot standby within the next 6 hours.
- C. (1) The symbol (**) is presented on the RM-11 screen indicating Channel Check Source Failed
(2) Initiate the preplanned alternate method of monitoring the appropriate parameter(s) within 72 hours.
- D. (1) The symbol (**) is presented on the RM-11 screen indicating Channel Check Source Failed
(2) Restore the PVS WRGM to operable status within 72 hours or be in hot standby within the next 6 hours.

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98. Given the following plant conditions:

- A Site Area Emergency has been declared
- The Technical Support Center has NOT yet been staffed
- The Shift Manager is acting as the Site Emergency Coordinator - Main Control Room (SEC-MCR) and has directed a team of two Maintenance personnel to work in a radiologically hazardous area of the RAB
- The Radiological Control Director (RCD) is standing by to assist
- Each person performing the work is expected to receive 5500 mrem TEDE for this entry

Which ONE of the following correctly describes the limitation basis for the type of entry AND the position that must approve each team member to receive the expected dose IAW PEP-330, Radiological Consequences?

<u>Limitation Basis</u>	<u>Position</u>
A. Protection of large populations ONLY	SEC-MCR
B. Protection of large populations ONLY	RCD
C. Protect valuable property or large populations	SEC-MCR
D. Protect valuable property or large populations	RCD

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99. Given the following plant conditions:

- The plant is operating at 100% power

The following occur

- A fire is reported in the 'A' Transfer panel
- The fire brigade is fighting the fire using water and the fire continues to burn
- The RO reports that the 'A' Group of PZR Backup Heaters is cycling ON and OFF
- The crew enters FPP-002, Fire Emergency, and AOP-036, Safe Shutdown

Following A Fire

- The CRS is evaluating if a normal plant shutdown can be achieved with at least one train of equipment using normal plant procedures

Which ONE of the following describes the action that will be taken with regards to AOP-036 (and it's subset procedures) and the action required for the 'A' Group of PZR Backup Heaters IAW FPP-002?

- A. Exit AOP-036 and go to procedure and step in effect
OPEN the feeder breaker to Bus 1A1
- B. Exit AOP-036 and go to procedure and step in effect
Rack out the breaker for the 'A' Group of PZR Backup Heaters
- C. Remain in AOP-036
Rack out the breaker for the 'A' Group of PZR Backup Heaters
- D. Remain in AOP-036
OPEN the feeder breaker to Bus 1A1

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100. Given the following plant conditions:

- A Reactor trip has occurred due to a loss of MFW
- FRP-H.1, Response to Loss of Secondary Heat Sink, is being implemented
- The RCS is in a bleed and feed condition
- RCS Tavg is 570°F and stable
- The operators have restored a feedwater source and are preparing to feed the SGs which are dry
- The CRS directs the operator to establish feed water to only one SG

Which ONE of the following describes the reason for feeding only one SG under these conditions?

- A. To prevent a rapid cooldown of the RCS that could lead to a pressurized thermal shock condition.
- B. To demonstrate the reliability of the FW source before filling all of the SGs.
- C. To determine if one SG is capable of maintaining adequate heat sink so that RCS bleed and feed can be terminated.
- D. To ensure that if a SG failure occurs due to excessive thermal stresses, the failure is isolated to one SG.

You have completed the test!

Harris 2011-301 List of References

RO Exam:

Steam Tables

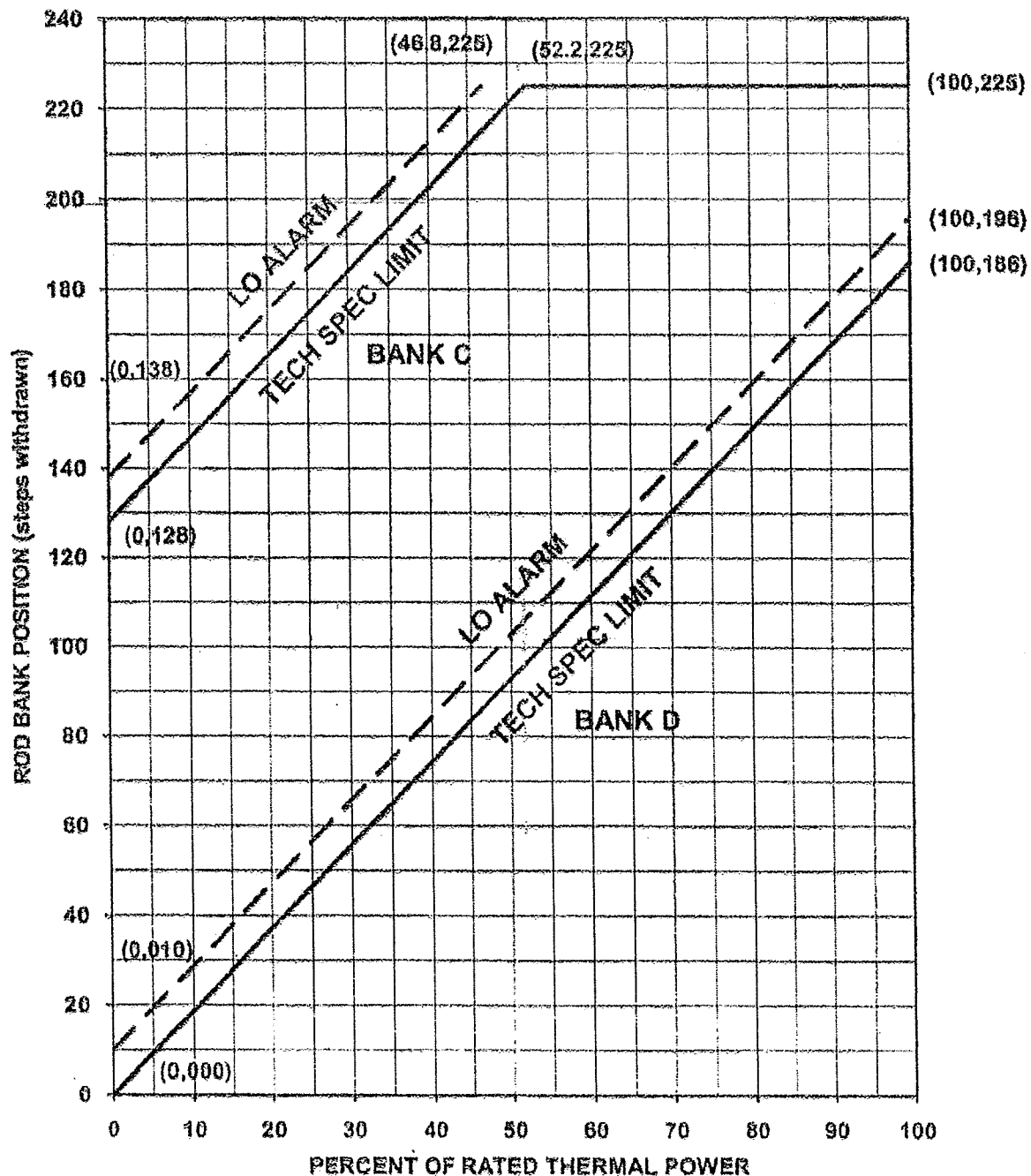
SRO Exam:

Steam Tables

Curve F-17-1, Revision 0. (Rod Insertion Limit Curve)

AOP-001 Revision 35, page 45, attachment 4

HARRIS UNIT 1 CYCLE 17 ROD INSERTION LIMITS

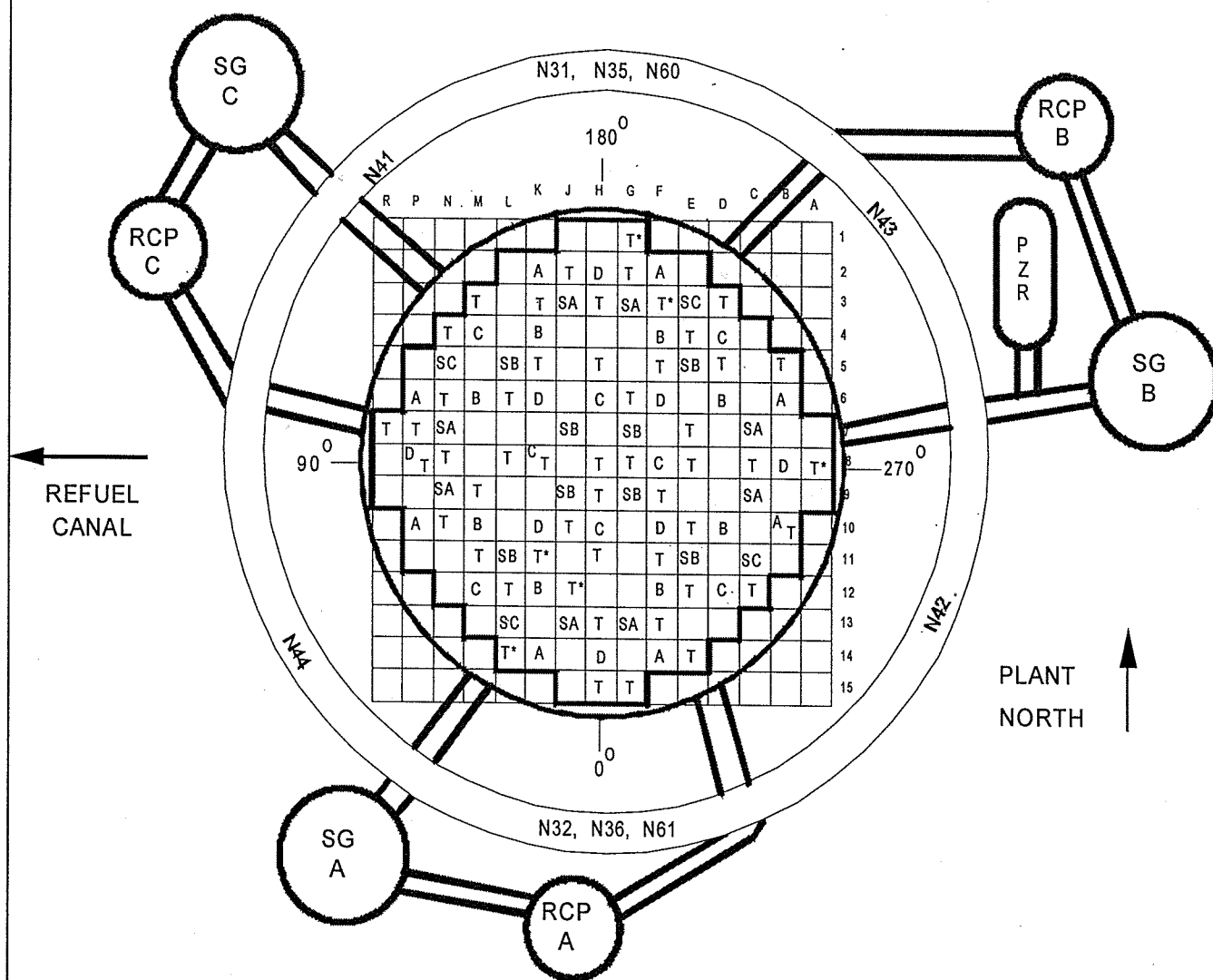


CURVE NO.	F-17-1	REV NO.	0
ORIGINATOR	<i>[Signature]</i>	DATE	10/21/10
SUPERVISOR	<i>[Signature]</i>	DATE	10/24/10
SUPERINTENDENT - SHIFT OPERATIONS	<i>[Signature]</i>	DATE	10-25-10

MALFUNCTION OF ROD CONTROL AND INDICATION SYSTEM

Attachment 4 – Core and RCS Overview

Sheet 1 of 1



T - Thermocouple

SA - Shutdown Bank A Rod

SB - Shutdown Bank B Rod

SC - Shutdown Bank C Rod

A - Control Bank A Rod

B - Control Bank B Rod

C - Control Bank C Rod

D - Control Bank D Rod

T* - thermocouples abandoned by EC 47997 (core locations A8, F3, G1, J12, K11, L14)

--END OF ATTACHMENT 4--

ANSWER KEY REPORT
for 2011 HNP SRO NRC Written Exam Test Form: 0

				Answers
#	ID	Points	Type	0
1	2011 NRC RO 1	1.00	MCS	A
2	2011 NRC RO 2	1.00	MCS	D
3	2011 NRC RO 3	1.00	MCS	A
4	2011 NRC RO 4	1.00	MCS	D
5	2011 NRC RO 5	1.00	MCS	A
6	2011 NRC RO 6	1.00	MCS	D
7	2011 NRC RO 7	1.00	MCS	A
8	2011 NRC RO 8	1.00	MCS	B
9	2011 NRC RO 9	1.00	MCS	A
10	2011 NRC RO 10	1.00	MCS	A
11	2011 NRC RO 11	1.00	MCS	C
12	2011 NRC RO 12	1.00	MCS	D
13	2011 NRC RO 13	1.00	MCS	A
14	2011 NRC RO 14	1.00	MCS	C
15	2011 NRC RO 15	1.00	MCS	B
16	2011 NRC RO 16	1.00	MCS	B
17	2011 NRC RO 17	1.00	MCS	B
18	2011 NRC RO 18	1.00	MCS	B
19	2011 NRC RO 19	1.00	MCS	D
20	2011 NRC RO 20	1.00	MCS	B
21	2011 NRC RO 21	1.00	MCS	C
22	2011 NRC RO 22	1.00	MCS	D
23	2011 NRC RO 23	1.00	MCS	D
24	2011 NRC RO 24	1.00	MCS	B
25	2011 NRC RO 25	1.00	MCS	C
26	2011 NRC RO 26	1.00	MCS	A
27	2011 NRC RO 27	1.00	MCS	C
28	2011 NRC RO 28	1.00	MCS	D
29	2011 NRC RO 29	1.00	MCS	A
30	2011 NRC RO 30	1.00	MCS	D
31	2011 NRC RO 31	1.00	MCS	D
32	2011 NRC RO 32	1.00	MCS	B
33	2011 NRC RO 33	1.00	MCS	C
34	2011 NRC RO 34	1.00	MCS	C
35	2011 NRC RO 35	1.00	MCS	B
36	2011 NRC RO 36	1.00	MCS	A
37	2011 NRC RO 37	1.00	MCS	C
38	2011 NRC RO 38	1.00	MCS	A
39	2011 NRC RO 39	1.00	MCS	D
40	2011 NRC RO 40	1.00	MCS	A
41	2011 NRC RO 41	1.00	MCS	D
42	2011 NRC RO 42	1.00	MCS	D
43	2011 NRC RO 43	1.00	MCS	D
44	2011 NRC RO 44	1.00	MCS	A
45	2011 NRC RO 45	1.00	MCS	D
46	2011 NRC RO 46	1.00	MCS	C
47	2011 NRC RO 47	1.00	MCS	B

ANSWER KEY REPORT

for 2011 HNP SRO NRC Written Exam Test Form: 0

				Answers
#	ID	Points	Type	0
48	2011 NRC RO 48	1.00	MCS	D
49	2011 NRC RO 49	1.00	MCS	A
50	2011 NRC RO 50	1.00	MCS	A
51	2011 NRC RO 51	1.00	MCS	D
52	2011 NRC RO 52	1.00	MCS	D
53	2011 NRC RO 53	1.00	MCS	C
54	2011 NRC RO 54	1.00	MCS	D
55	2011 NRC RO 55	1.00	MCS	C
56	2011 NRC RO 56	1.00	MCS	A
57	2011 NRC RO 57	1.00	MCS	C
58	2011 NRC RO 58	1.00	MCS	A
59	2011 NRC RO 59	1.00	MCS	D
60	2011 NRC RO 60	1.00	MCS	A
61	2011 NRC RO 61	1.00	MCS	A
62	2011 NRC RO 62	1.00	MCS	B
63	2011 NRC RO 63	1.00	MCS	A
64	2011 NRC RO 64	1.00	MCS	D
65	2011 NRC RO 65	1.00	MCS	C
66	2011 NRC RO 66	1.00	MCS	A
67	2011 NRC RO 67	1.00	MCS	C
68	2011 NRC RO 68	1.00	MCS	B
69	2011 NRC RO 69	1.00	MCS	B
70	2011 NRC RO 70	1.00	MCS	D
71	2011 NRC RO 71	1.00	MCS	C
72	2011 NRC RO 72	1.00	MCS	B
73	2011 NRC RO 73	1.00	MCS	D
74	2011 NRC RO 74	1.00	MCS	C
75	2011 NRC RO 75	1.00	MCS	A
76	2011 NRC SRO 1	1.00	MCS	B
77	2011 NRC SRO 2	1.00	MCS	C
78	2011 NRC SRO 3	1.00	MCS	C
79	2011 NRC SRO 4	1.00	MCS	A
80	2011 NRC SRO 5	1.00	MCS	B
81	2011 NRC SRO 6	1.00	MCS	A
82	2011 NRC SRO 7	1.00	MCS	A
83	2011 NRC SRO 8	1.00	MCS	B
84	2011 NRC SRO 9	1.00	MCS	D
85	2011 NRC SRO 10	1.00	MCS	A
86	2011 NRC SRO 11	1.00	MCS	D
87	2011 NRC SRO 12	1.00	MCS	A
88	2011 NRC SRO 13	1.00	MCS	D
89	2011 NRC SRO 14	1.00	MCS	C
90	2011 NRC SRO 15	1.00	MCS	C
91	2011 NRC SRO 16	1.00	MCS	A
92	2011 NRC SRO 17	1.00	MCS	B
93	2011 NRC SRO 18	1.00	MCS	A
94	2011 NRC SRO 19	1.00	MCS	C

ANSWER KEY REPORT
for 2011 HNP SRO NRC Written Exam Test Form: 0

				Answers
#	ID	Points	Type	0
95	2011 NRC SRO 20	1.00	MCS	D
96	2011 NRC SRO 21	1.00	MCS	D
97	2011 NRC SRO 22	1.00	MCS	C
98	2011 NRC SRO 23	1.00	MCS	C
99	2011 NRC SRO 24	1.00	MCS	D
100	2011 NRC SRO 25	1.00	MCS	D
SECTION 1 (100 items)		100.00		