



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
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ATLANTA, GEORGIA 30303-1257

August 4, 2011

L. Mike Stinson
Interim Vice President - Farley
Southern Nuclear Operating Company, Inc.
7388 North State Highway 95
Columbia, AL 36319

**SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT - NRC INTEGRATED INSPECTION
REPORT 05000348/2011003 AND 05000364/2011003**

Dear Mr. Stinson:

On June 30, 2011, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your Joseph M. Farley Nuclear Plant, Units 1 and 2. The enclosed inspection report documents the inspection results, which were discussed on July 14, 2011, with members of your staff.

The inspection examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations, and with the conditions of your license. The NRC reviewed selected procedures and records, observed activities and interviewed personnel. Based on the results of this inspection, no findings were identified.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response, if any, will be available electronically for public inspection in the NRC public document room or from the publicly available records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC website at <http://www.nrc.gov/reading-rm/adams.html> (the public electronic reading room).

Sincerely,

/RA/

Scott M. Shaeffer, Chief
Reactor Projects Branch 2
Division of Reactor Projects

Docket No.: 50-348, 50-364
License No.: NPF-2, NPF-8

Enclosure: Inspection Report 05000348/2011003 and 05000364/2011003
w/Attachment: Supplemental Information

cc w/encl.: (See page 3)

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Letter to L. Mike Stinson from Scott M. Shaeffer dated August 4, 2011

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT - NRC INTEGRATED INSPECTION
REPORT 05000348/2011003 AND 05000364/2011003

Distribution w/encl:

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U.S. NUCLEAR REGULATORY COMMISSION

REGION II

Docket Nos.: 05000348, 05000364

License Nos.: NPF-2, NPF-8

Report No.: 05000348/2011003 and 05000364/2011003

Licensee: Southern Nuclear Operating Company, Inc.

Facility: Joseph M. Farley Nuclear Plant, Units 1 and 2

Location: Columbia, AL

Dates: April 1, 2011, through June 30, 2011

Inspectors: E. Crowe, Senior Resident Inspector
J. Sowa, Resident Inspector
R. Patterson, Physical Security Inspector (Section 4OA5.2)
R. Russell, Resident Inspector, Paducah (Sections 1EP2-6 and 4OA1)
M. Speck, Senior Emergency Preparedness Inspector (Sections 1EP2-6 and 4OA1)

Approved by: Scott M. Shaeffer, Chief
Reactor Projects Branch 2
Division of Reactor Projects

Enclosure

SUMMARY OF FINDINGS

IR 05000348/2011003 and 05000364/2011003; April 1, 2011, through June 30, 2011; Joseph M. Farley Nuclear Plant, Units 1 and 2; Routine Integrated Report.

The report covered a three-month period of inspection by the resident inspectors and two regional inspectors. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 4, dated December, 2006. No findings were identified during this inspection period.

A. NRC-Identified and Self-Revealing Findings

No findings were identified.

B. Licensee-identified Violations

None

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REPORT DETAILS

Summary of Plant Status

Unit 1 started the report period at 100 percent rated thermal power (RTP). On April 29, 2011, the unit was ramped to 60 percent RTP for repairs to an oil leak on the 1B steam generator (SG) feed pump. The unit returned to 100 percent RTP on April 30, 2011. On May 21, 2011, the unit was ramped to 61 percent RTP for planned repairs to the 1B SG feed pump controller power supply. The unit returned to 100 percent RTP later that day, and remained at or near 100 percent RTP for the remainder of the inspection period.

Unit 2 started the report period at 100 percent RTP. On April 8, 2011, the unit was ramped to 59 percent RTP to seal steam leaks on the moisture separator reheat shell drain tank. The unit returned to 100 percent RTP on April 12, 2011. On June 18, 2011, the 2A circulating water pump tripped, forcing control room operators to reduce power to 71 percent RTP. The unit returned to 100 percent RTP on June 21, 2011, and remained at or near 100 percent RTP for the remainder of the inspection period.

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, Barrier Integrity

1R01 Adverse Weather Protection

a. Inspection Scope

Offsite/Alternate AC Readiness: The inspectors reviewed the licensee's station procedures to verify communication protocols existed between the transmission operator and control room to promptly identify issues impacting the offsite power system. The inspectors verified procedure adequacy to address measures to monitor and maintain availability and reliability of the offsite alternating current (AC) power system (Alabama Power Company) and the onsite alternate AC power system. The inspectors also reviewed the compensatory actions identified in station procedures to be performed when it is not possible to predict post-trip voltage at the site for current electrical grid conditions. Documents reviewed are listed in the Attachment.

b. Findings

No findings were identified.

1R04 Equipment Alignment

a. Inspection Scope

Partial Walk-Down: The inspectors performed partial walk-downs of the following three systems to verify operability of redundant or diverse trains and components when safety equipment was inoperable. The inspectors attempted to identify discrepancies impacting the function of the system and therefore, potentially increasing risk. The walk-downs were performed using the criteria in licensee procedures NMP-OS-007, Conduct

Enclosure

of Operations, and FNP-0-SOP-0, General Instructions to Operations Personnel. The walk-downs included reviewing the updated final safety analysis report (UFSAR), plant procedures and drawings, checks of control room and plant valves, switches, components, electrical power, support equipment and instrumentation. Documents reviewed are listed in the Attachment.

- Unit 1, A train motor driven auxiliary feedwater pump (MDAFWP) while B train was out of service (OOS)
- Unit 1 and Unit 2, A train residual heat removal (RHR) systems during planned maintenance to B train pumps
- Unit 2, A train component cooling water (CCW) while B train was OOS

Complete Walk-Down: The inspectors conducted a complete walk-down of the accessible portions of the following system. The inspectors used licensee procedure FNP-1-SOP-7.0, RHR System, and Functional System Description (FSD) A181002, to verify system alignment of on-service equipment. The inspectors also interviewed personnel, reviewed control room logs, maintenance rule (MR) monthly reports, condition reports (CRs), quarterly system health reports, outstanding work orders (WOs) and industry operating experience (OE) to verify alignment and equipment discrepancies were being identified and appropriately resolved. Documents reviewed are listed in the Attachment.

- Unit 1 RHR system

b. Findings

No findings were identified.

1R05 Fire Protection

a. Inspection Scope

Fire Protection Area Tours: The inspectors conducted a tour of the four fire areas listed below to assess material condition and operational status of the fire protection equipment. The inspectors verified combustibles and ignition sources were controlled in accordance with the licensee's administrative procedures; fire detection and suppression equipment was available for use; passive fire barriers were maintained in good material condition and compensatory measures for out-of-service, degraded, or inoperable fire protection equipment were implemented in accordance with the requirements of licensee procedures FNP-0-AP-36, Fire Surveillance and Inspection; FNP-0-AP-38, Use of Open Flame; FNP-0-AP-39, Fire Patrols and Watches; and the associated fire zone data sheets. Documents reviewed are listed in the Attachment.

- Unit 1, RHR heat exchanger room, fire zone 1
- Unit 1, spent fuel pool (SFP) room, fire zone 4
- Units 1 and 2, control room ventilation rooms, fire zone 51
- Unit 2, B train battery room, fire zone 16

b. Findings

No findings were identified.

1R11 Licensed Operator Regualification Program

.1 Resident Inspector Quarterly Review:

a. Inspection Scope:

On May 3, 2011, the inspectors observed portions of the licensed operator training and testing program to verify implementation of procedures FNP-0-AP-45, Farley Nuclear Plant Training Plan; FNP-0-TCP-17.6, Simulator Training Evaluation/Documentation; and FNP-0-TCP-17.3, Licensed Operator Continuing Training Program Administration. The inspectors observed operations simulator scenario 11-S0602, conducted in the licensee's simulator for a main steam break, resulting in a rapid ramp-down of reactor power, failures of on-service CCW and charging pumps, a malfunctioning of charging letdown malfunction and a reactor trip as a result of the worsening main steam break. The inspectors observed high-risk operator actions, overall crew performance, self-critiques, training feedback and management oversight to verify operator performance was evaluated against the performance standards of the licensee's scenario. Documents reviewed are listed in the Attachment.

b. Findings

No findings were identified.

.2 Operating Experience Smart Sample (OpESS) FY 2010-02 "Sample Selections for Reviewing Licensed Operator Examinations and Training Conducted on the Plant-Referenced Simulator."

a. Inspection Scope:

The inspectors conducted interviews with operations management, training department staff and licensed operators to determine the type and content of training performed related to complex transients and trips. The inspectors also discussed the inclusion of operating experience related to fires in electrical equipment similar to an event occurring at the H. B. Robinson Plant on March 28, 2010. The inspectors attended plant meetings and observed the presentation of operating experience, which included the H. B. Robinson Plant event. The inspectors obtained and reviewed a copy of the licensee's operator requalification training scenario, designed to mimic the H.B. Robinson Plant electrical fire event. The licensee conducted this scenario for all operations department crews during cycle four (4) training in 2010.

b. Findings

No findings were identified.

1R12 Maintenance Rule Effectiveness.1 Resident Inspector Quarterly Inspection Samplesa. Inspection Scope

The inspectors reviewed the following two activities for (1) appropriate work practices; (2) identifying and addressing common cause failures; (3) scoping in accordance with 10 CFR 50.65(b) of the MR; (4) characterizing reliability issues for performance; (5) trending key parameters for condition monitoring; (6) charging unavailability for performance; (7) classification and reclassification in accordance with 10 CFR 50.65(a)(1) or (a)(2); and (8) appropriateness of performance criteria for structures, systems, and components (SSCs)/functions classified as (a)(2) and/or appropriateness and adequacy of goals and corrective actions for SSCs/functions classified as (a)(1). In addition, the NRC specifically reviewed events where ineffective equipment maintenance resulted in invalid automatic actuations of engineered safeguards systems affecting the operating units. Documents reviewed are listed in the Attachment.

- CR 2010110046, Q2HV3234B, turbine driven auxiliary feedwater pump (TDAFWP) warm up valve failed to stroke fully closed
- CR 2011105288, technical support center (TSC) HVAC repairs

b. Findings

No findings were identified.

.2 OpESS FY2010-01 "Recent Inspection Experience for Components Installed Beyond Vendor-Recommended Service Life"a. Inspection Scope

The inspectors conducted interviews with operations, maintenance and engineering staff to determine the licensee's method of monitoring the service life of plant-installed components. The inspectors reviewed the licensee's procedure NMP-ES-003, Life Cycle Management, Version 11.0, to evaluate how the licensee implemented vendor-recommended service life for station components. The inspectors also conducted interviews with the licensee's design modification department staff to identify their process for new equipment used in plant modifications.

b. Findings

No findings were identified.

1R13 Maintenance Risk Assessments and Emergent Work Evaluation

a. Inspection Scope

The inspectors reviewed the following four activities to verify appropriate risk assessments were performed prior to taking equipment OOS for maintenance. The inspectors verified risk assessments were performed as required by 10 CFR 50.65(a)(4), and were accurate and complete. When emergent work was performed, the inspectors verified appropriate use of the licensee's risk assessment and risk categories in accordance with requirements in licensee procedures FNP-0-ACP-52.3, Mode 1, 2, & 3 Risk Assessment; FNP-0-UOP-4.0, General Outage Operations Guidance; NMP-GM-006, Work Management; and NMP-OS-007, Conduct of Operations.

- April 13, 2011 Unit 2, YELLOW risk condition concurrent with unavailability of the 2A MDAFWP
- May 6, 2011, Unit 1, YELLOW risk condition with concurrent unavailability of the 1B MDAFWP
- May 9, 2011, Unit 1, YELLOW risk condition with concurrent unavailability of the 1B SFP heat exchanger
- June 7, 2011, Unit 1, YELLOW risk condition concurrent with unavailability of the 1A MDAFWP and surveillance of the 1C emergency diesel generator (EDG)

b. Findings

No findings were identified.

1R15 Operability Evaluations

a. Inspection Scope

The inspectors reviewed the following seven operability evaluations to verify the requirements of licensee procedures NMP-OS-007, Conduct of Operations, and NMP-AD-012, ODs and Functionality Assessments, were met. The scope of this inspection also included a review of the technical adequacy of the evaluations, the adequacy of compensatory measures and the impact on continued plant operation.

- CR 2011105276, computer room air conditioning unit shutdown during troubleshooting of computer room Halon system
- CR 2011105597, 2B component cooling water (CCW) pump did not start when the start switch was operated from the main control board
- CR 2011106318, 2A battery charger failed to swap from float to equalize
- CR 2011106641, 1C EDG oil temperature high (OTH) and coolant temperature high (CTH) temperature switches not terminated in accordance with approved design
- CR 2011106792, CCW supply valve to 1A RHR heat exchanger was demanded closed with no appropriate change in flow parameters

- CR 2011107041, CCW leak on 1B CCW pump mini-flow line
- CR 2011107156, lower than expected flow to SW pond when unit 2 service water (SW) recirculation was established

b. Findings

No findings were identified.

1R18 Plant Modifications

a. Inspection Scope

The inspectors reviewed the following plant modification to ensure the safety functions of important safety systems were unaffected. The inspectors also verified design bases, licensing bases and performance capability of risk-significant SSCs had not been degraded through modifications. The inspectors verified any modification performed during risk-significant configuration did not place the plant in an unsafe condition. The inspectors evaluated system operability, availability, configuration control, post-installation test activities, documentation updates and operator awareness of the modification. Documents reviewed are listed in the Attachment.

- TM 1111087101, temporary mounting and connection of a dual power supply for the 1A SG feed pump electro-hydraulic controller

b. Findings

No findings were identified.

1R19 Post Maintenance Testing

a. Inspection Scope

The inspectors reviewed the criteria contained in licensee procedures FNP-0-PMT-0.0, Post-Maintenance Test Program, to verify post-maintenance test procedures and test activities for the following five systems/components were adequate to verify system operability and functional capability. The inspectors also witnessed the test or reviewed the test data to verify test results adequately demonstrated restoration of the affected safety functions. Documents reviewed are listed in the Attachment.

- FNP-1-STP-4.2, 1B Charging Pump Quarterly Inservice Test (IST), following maintenance to the 1B charging pump electrical breaker
- FNP-1-STP-10.5, Emergency Boration Valve Quarterly Inservice Test following troubleshooting activities to establish reason for loss of position indication
- FNP-2-STP-24.1, 2A, 2B, and 2C Service Water Pump (SWP) Quarterly Inservice Test following repairs to 2B SWP discharge check valve
- Maintenance and troubleshooting of 2B CCW handswitch after pump failed to start
- Leakage test of the 1B SFP heat exchanger following repairs to leaking tube plugs

b. Findings

No findings were identified.

1R22 Surveillance Testing

a. Inspection Scope

The inspectors reviewed the following seven surveillance tests and either observed the test or reviewed test results to verify testing adequately demonstrated equipment operability and met technical specifications (TS) requirements. The inspectors reviewed the activities to assess for preconditioning of equipment, procedure adherence and valve alignment following completion of the surveillance. The inspectors reviewed licensee procedures FNP-0-AP-24, Test Control; FNP-0-M-050, Master List of Surveillance Requirements; and NMP-OS-007, Conduct of Operations, and attended selected briefings to determine if procedure requirements were met. Documents reviewed are listed in the Attachment.

Surveillance Tests

- FNP-1-STP-16.1, 1A Containment Spray Pump Quarterly Inservice Test
- FNP-1-STP-913.0, Reactor Coolant Pump (RCP) Bus Reactor Trip Underfrequency Relay Test
- FNP-1-STP-912.1, Reactor Coolant Pump Bus Reactor Trip Undervoltage TDAFW Start Relay Test
- FNP-2-STP-201.19, RCS Q2B13TE0422B1, Q2B13TE0422B2, Q2B13TE0422B3, and Q2B13TE0422D Loop Calibration and Operational Test

In-Service Test (IST)

- FNP-1-STP-16.1, 1A Containment Spray Pump Quarterly Inservice Test

Reactor Coolant System (RCS) Leak Detection

- FNP-1-STP-9.0, RCS Leakage Test
- FNP-2-STP-9.0, RCS Leakage Test

b. Findings

No findings were identified.

Cornerstone: Emergency Preparedness (EP)

1EP2 Alert and Notification System (ANS) Testing

a. Inspection Scope

The inspectors evaluated the adequacy of the licensee's operation, maintenance and periodic testing of the Alert and Notification System (ANS) using NRC IP 71114, Attachment 02, Alert and Notification System Evaluation. The inspectors gathered

information through document reviews and interviews, and reviewed monthly trend reports and siren test failure records. Additionally, the inspectors observed a weekly siren silent test and tone alert radio (TAR) test conducted from the Houston County Emergency Management Agency (EMA) Emergency Operations Center (EOC), to verify the test was conducted in accordance with the approved procedures. The inspectors used the applicable planning standard of 10 CFR Part 50.47(b)(5) and the related requirements in 10 CFR Part 50, Appendix E, Section IV.D, as reference criteria. The criteria contained in NUREG-0654, Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants, Revision 1, was also used as a reference.

The inspectors reviewed various documents as listed in the Attachment. The inspection activity satisfied one inspection sample for the ANS on a biennial basis as defined in IP 71114.02-05.

b. Findings

No findings were identified.

1EP3 Emergency Preparedness Organization Staffing and Augmentation System

a. Inspection Scope

The inspectors reviewed the licensee's Emergency Response Organization (ERO) augmentation staffing requirements and process for notifying the ERO to ensure the readiness of key staff for responding to an event and timely facility activation. The qualification records of key position ERO personnel were reviewed to ensure all ERO qualifications were current. A sample of problems identified from unannounced off-hour augmentation drills or system tests performed since the last inspection were reviewed to assess the effectiveness of corrective actions. The inspectors reviewed the records from the unannounced off-hour recall drill conducted August 12, 2009, to verify the ERO members were alerted and mobilized and the response facilities were staffed and activated in a timely manner.

The inspection was conducted in accordance with NRC IP 71114, Attachment 03, Emergency Response Organization Staffing and Augmentation System. The inspectors used the applicable planning standard of 10 CFR 50.47(b)(2), and the related requirements in 10 CFR 50, Appendix E, as reference criteria.

The inspectors reviewed various documents as listed in the Attachment. The inspection activity satisfied one inspection sample for the ERO staffing and augmentation system on a biennial basis as defined in IP 71114.03-05.

b. Findings

No findings were identified.

1EP4 Emergency Action Level and Emergency Plan Changes

a. Inspection Scope

Since the last NRC inspection, the licensee implemented revision 52 of the Joseph M. Farley Nuclear Plant Emergency Plan. The inspectors conducted a review of the emergency action level changes and sampled the revisions to the emergency plan and the implementing procedure made between December 1, 2010, and April 30, 2011, to evaluate the changes identified in the revisions for potential decreases in effectiveness of the emergency plan. The inspection included a review of the 10 CFR 50.54(q) change process documentation. The licensee determined the changes resulted in no decrease in the effectiveness of the emergency plan and the revised plan continued to meet the requirements of 10 CFR 50.47(b) and Appendix E to 10 CFR Part 50. The NRC review of the revisions does not constitute formal approval of the changes and was not documented in a safety evaluation report; therefore, the emergency action level and emergency plan changes remain subject to future NRC inspection in their entirety.

The inspection was conducted in accordance with NRC Inspection Procedure 71114, Attachment 04, Emergency Action Level and Emergency Plan Changes. The inspectors used the applicable planning standard of 10 CFR 50.47(b)(4) and the related requirements in 10 CFR 50, Appendix E, as reference criteria.

The inspectors reviewed various documents listed in the Attachment. The inspection activity satisfied one inspection sample for the emergency action level and emergency plan changes on an annual basis as defined in IP 71114.04-05.

b. Findings

No findings were identified.

1EP5 Correction of Emergency Preparedness (EP) Weaknesses and Deficiencies

a. Inspection Scope

The inspectors reviewed the corrective actions identified through the EP program to determine the significance of the issues and to evaluate the licensee's efforts to identify, track and resolve deficiencies. The inspectors reviewed the fleet oversight audits and assessments of the EP program to determine if the independent assessments met the requirements of 10 CFR 50.54(t). The inspectors also reviewed critique reports and samples of CAP records associated with the 2010 biennial exercise, as well as various EP drills conducted in 2010, to determine if the licensee fulfilled drill commitments and to assess the completeness and effectiveness of related corrective actions.

The inspection was conducted in accordance with NRC Inspection Procedure 71114, Attachment 05, Correction of Emergency Preparedness Weaknesses. The inspectors used the applicable planning standard of 10 CFR 50.47(b)(14) and the related requirements in 10 CFR 50, Appendix E, as reference criteria.

The inspectors reviewed documents which are listed in the Attachment. The inspection activity satisfied one inspection sample for the correction of EP weaknesses on a biennial basis as defined in IP 71114.05-05.

b. Findings

No findings were identified.

1EP6 Drill Evaluation

a. Inspection Scope

The NRC evaluated the conduct of routine licensee emergency drills on the following date to identify any weaknesses and deficiencies in classification, notification and protection action recommendation (PAR) development activities. The NRC observed emergency response operation in a simulated control room to verify event classification and notifications were performed in accordance with FNP-0-EIP-9.0, Emergency Classification and Actions. The NRC used procedure FNP-0-EIP-15.0, Emergency Drills, as the inspection criteria. The NRC also evaluated the licensee critique of the drill to compare any inspector-observed weaknesses with those identified by the licensee in order to verify whether the licensee was properly identifying issues.

- May 25, 2011 – General Emergency due to containment leak to the lower equipment room coincident with a 1000 gallon per minute RCS leak and fuel cladding failure.

b. Findings

No findings were identified.

4. OTHER ACTIVITIES

4OA1 Performance Indicator (PI) Verification

a. Inspection Scope

The inspectors sampled licensee data for the performance indicators (PIs) listed below to verify the accuracy of the PI data reported during the period listed. For the emergency preparedness cornerstone, the period included the fourth quarter 2010 through first quarter 2011. Nuclear Energy Institute (NEI) 99-02, Regulatory Assessment Indicator Guideline, Rev. 6, was used to verify the basis in reporting for each data element. Documents reviewed are listed in the Attachment.

Cornerstone: Initiating Events

- Unplanned scrams with complications

Cornerstone: Mitigating Systems

- Mitigating systems performance indicator (MSPI), heat removal system
- MSPI, emergency AC power system

The inspectors reviewed samples of raw PI data, licensee event reports, and monthly operating reports for the period covering March, 2010 through March, 2011. The data reviewed was compared to graphical representations from the most recent PI report. The inspectors also examined a sampling of operations logs and procedures to verify PI data was appropriately captured for inclusion into the PI report, as well as ensuring the individual PIs were calculated correctly.

Cornerstone: Emergency Preparedness

- Alert and Notification System Reliability (ANS)

The inspection was conducted in accordance with NRC Inspection Procedure 71151, Performance Indicator Verification. For the specified review period, the inspectors examined data reported to the NRC, procedural guidance for reporting PI information, and records used by the licensee to identify potential PI occurrences.

For the ANS indicator, the inspector reviewed of a sample of the licensee's records of periodic system tests. The inspectors reviewed the records of the licensee's reported number of successful siren operability tests as compared to the number of siren tests conducted during the reporting period, to validate the accuracy of the PI submittals.

This inspection activity satisfied one inspection sample for the Alert and Notification System as defined in IP 71151-05.

b. Findings

No findings were identified.

4OA2 Identification and Resolution of Problems

.1 Daily CR Reviews

As required by IP 71152, Identification and Resolution of Problems, and in order to help identify repetitive equipment failures or specific human performance issues for follow-up, the NRC performed a daily screening of items entered into the licensee's corrective action program (CAP). This review was accomplished by reviewing hard copies of CRs, attending daily screening meetings and accessing the licensee's computerized database.

.2 Selected Issue Follow-up Inspection

a. Inspection Scope

In addition to the routine review, the inspectors selected the one issue listed below for a more in-depth review. The inspectors considered the following during the review of the licensee's actions: (1) complete and accurate identification of the problem in a timely manner; (2) evaluation and disposition of operability/reportability issues; (3) consideration of extent of condition, generic implications, common cause, and previous occurrences; (4) classification and prioritization of the resolution of the problem; (5) identification of root and contributing causes of the problem; (6) identification of CRs; and (7) completion of corrective actions in a timely manner.

- CR 2011103701, trip of 1C EDG during 24 hour surveillance run

b. Findings

No findings were identified.

.3 Semi-Annual Trend Review

a. Inspection Scope

As required by IP 71152, Identification and Resolution of Problems, the inspectors performed a review of the licensee's CAP and associated documents to identify trends that could indicate the existence of a more significant safety issue. The inspectors reviewed repetitive equipment and corrective maintenance issues, and also considered the results of daily inspector CAP item-screening discussed above. The review also included issues documented outside the normal CAP process, including system health reports, corrective maintenance WOs, component status reports and MR assessments. The inspectors' review nominally considered the six-month period of January 1 through June 30, 2011, although some examples expanded beyond those dates when the scope of the trend warranted. The inspectors compared and contrasted their results with the results contained in the licensee's latest integrated quarterly assessment report. Corrective actions associated with the sample of the issues identified in the licensee's trend report were reviewed for adequacy. Documents reviewed are listed in the Attachment.

b. Assessment and Observations

The inspectors evaluated performance data provided by the Farley performance improvement group for the period of time from January 1 through April 30, 2011. The inspectors noted the continued licensee focus on procedural adherence and continued improvement in the trend of procedure use and adherence. The inspectors also noted the licensee's trend of identifying procedural issues continued to have the same low threshold, and station personnel continued writing CRs to identify needed procedural enhancements. In addition, the licensee's central procedure group has worked to reduce the backlog of open procedure items. The inspectors additionally noted an increase in

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the licensee's identification of inadequate corrective actions associated with CRs, although the inspectors did not discover any inadequate corrective actions with significant safety significance. The inspectors noted the licensee identified two new trends in its CAP, in the control of combustibles and quality of physical work. The inspectors did not identify any adverse trend from the licensee's CAP, or from their review of issues documented outside the normal CAP process, including system health reports, corrective maintenance WOs, component status reports, and MR assessments.

40A5 Other Activities

.1 Quarterly Resident Inspector Observations of Security Personnel and Activities

a. Inspection Scope

During the inspection period, the inspectors conducted observations of security force personnel and activities to ensure the activities were consistent with licensee security procedures and regulatory requirements relating to nuclear plant security. These observations took place during both normal and off-normal plant working hours. These quarterly resident inspector observations of security force personnel and activities did not constitute any additional inspection samples. Rather, they were considered an integral part of the inspectors' normal plant status reviews and inspection activities.

b. Findings

No findings were identified.

.2 Protected Area (PA) Expansion

a. Inspection Scope

June 29 through July 1, 2011, an NRC Region 2 security inspector performed a site visit to review and observe the licensee's PA expansion. The inspector evaluated the licensee's security force training, change management and action plan to implement the changes to the PA boundaries and PA personnel access point (PAP). The inspector observed the licensee's searches and sanitization of the new PA area to include all buildings incorporated into the PA. The inspector attended security force shift briefings and walked down the new PA boundary, intrusion detection zones, and new security positions. These inspector observations of the licensee's PA expansion and associated activities did not constitute any additional inspection samples.

.3 (Closed) NRC Temporary Instruction 2515/183, "Follow-up to the Fukushima Daiichi Nuclear Station Fuel Damage Event"

a. Inspection Scope

The inspectors assessed the activities and actions taken by the licensee to assess its readiness to respond to an event similar to the Fukushima Daiichi nuclear plant fuel damage event. This included (1) an assessment of the licensee's capability to mitigate

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conditions that may result from beyond design basis events, with a particular emphasis on strategies related to the spent fuel pool, as required by NRC Security Order Section B.5.b issued February 25, 2002, as committed to in severe accident management guidelines, and as required by 10 CFR 50.54(hh); (2) an assessment of the licensee's capability to mitigate station blackout (SBO) conditions, as required by 10 CFR 50.63 and station design bases; (3) an assessment of the licensee's capability to mitigate internal and external flooding events, as required by station design bases; and (4) an assessment of the thoroughness of the walkdowns and inspections of important equipment needed to mitigate fire and flood events, which were performed by the licensee to identify any potential loss of function of this equipment during seismic events possible for the site.

b. Findings

Inspection Report 05000348,364/2011011 (ML111330097) documented detailed results of this inspection activity. Following issuance of the report, the inspectors conducted detailed follow-up on selected issues. No findings were identified during this follow-up inspection.

.4 (Closed) NRC Temporary Instruction 2515/184, "Availability and Readiness Inspection of Severe Accident Management Guidelines (SAMGs)"

a. Inspection Scope

On May 27, 2011, the inspectors completed a review of the licensee's severe accident management guidelines (SAMGs), implemented as a voluntary industry initiative in the 1990's, to determine (1) whether the SAMGs were available and updated, (2) whether the licensee had procedures and processes in place to control and update its SAMGs, (3) the nature and extent of the licensee's training of personnel on the use of SAMGs, and (4) licensee personnel's familiarity with SAMG implementation.

The results of this review were provided to the NRC task force chartered by the Executive Director for Operations to conduct a near-term evaluation of the need for agency actions following the Fukushima Daiichi fuel damage event in Japan. Plant-specific results for Farley Nuclear Plant were provided as an Enclosure to a memorandum to the Chief, Reactor Inspection Branch, Division of Inspection and Regional Support, dated June 02, 2011 (ML111530328).

b. Findings

No findings were identified.

4OA6 Meetings, Including Exit

The inspectors presented the interim emergency preparedness program inspection results onsite to T. Youngblood and other members of licensee management on May 12, 2011 and final inspection results by phone to J. Horn and other members of licensee management on June 3, 2011. The inspectors confirmed none of the potential report

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information discussed was considered proprietary. Proprietary material received during the inspection was returned to the licensee.

On July 14, 2011, the NRC presented the inspection results to you and members of your staff who acknowledged the results. The NRC confirmed proprietary information was not provided or examined during the inspection.

ATTACHMENT: SUPPLEMENTAL INFORMATION

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SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee personnel

C. Boone, Fleet EP Manager
B. Boyd, EP Specialist
D. Gilbert, EP Assistant
B. Griner, Engineering Director
J. Horn, Site Support Manager
F. Hundley, Fleet Oversight Supervisor
J. Jerkins, Corrective Action Program Supervisor
R. Martin, Engineering Programs Manager
S. Odom, Emergency Preparedness (EP) Supervisor
W. Oldfield, Licensing Engineer
T. Pelham, Performance Improvement Supervisor
D. Reed, Operations Support Superintendent
R. Roberson, Fleet Oversight
D. Simmons, EP Specialist
L. Smith, Maintenance Manager
S. Varnum, Chemistry Manager
C. Westberry, Engineering Systems Manager
T. Youngblood, Plant Manager

NRC personnel

Scott M. Shaeffer, Chief, Branch 2, Division of Reactor Projects

LIST OF REPORT ITEMS

Opened

None

Opened and Closed

None

Closed

2515/183	TI	Follow-up to the Fukushima Daiichi Nuclear Station Fuel Damage Event. (Section 4OA5.3)
2515/184	TI	Availability and Readiness Inspection of Severe Accident Management Guidelines (SAMGs). (Section 4OA5.4)

Discussed

None

LIST OF DOCUMENTS REVIEWED

Section 1R01: Adverse Weather Protection

Condition Reports:

2010106787, 2010107759

Procedures:

FNP-0-ACP-4.0, Switchyard Control, Version 13.0
FNP-0-AOP-21.0, Severe Weather, Version 30.0
FNP-1-AOP-5.2, Degraded Grid, Version 14.0
FNP-1-UOP-3.1, Power Operations, Version 110.0
FNP-2-AOP-5.2, Degraded Grid, Version 14.0
FNP-2-UOP-3.1, Power Operations, Version 99.0

Section 1R04: Equipment Alignment

Condition Reports:

2011101010, 2011103909, 2011104251, 2010114836, 2010114837, 2011105597, 2010115754,
2010115880, 2010116931

Drawings:

D175038, Sheet 1, Version 1.0
D175038, Sheet 2, Version 22.0
D175038, Sheet 3, Version 26.0
D175041, Sheet 1, Version 18.0

Procedures:

FNP-1-SOP-7.0, Residual Heat Removal System, Version 98.0
FNP-1-SOP-22.0, Auxiliary Feedwater System, Version 66.0
FNP-1-SOP-22.0A, Auxiliary Feedwater System, Version 10.0
FNP-2-SOP-23.0, Component Cooling Water, Version 88.0
FNP-2-SOP-23.0A, Component Cooling Water System, Version 11.0

Section 1R05: Fire Protection

A508650, Sheet 06, Aux Bldg. Elevation 83' – 0", Version 1.0
A508650, Sheet 34, Aux. Bldg. Elevation 155' – 0", Version 1.0
A508650, Sheet 45, Aux. Bldg. Elevation 175' – 0", Version 7.0
A509018, Sheet 20, Aux. Bldg. Elevation 121'- 0", Version 18.0

Section 1R11: Licensed Operator Regualification Program

Licensed Operator Continuing Training Simulator Exercise Guide: LOCT 10-12 Segment 6,
2011-S0602

Section 1R12: Maintenance Effectiveness

Action Items:

2009209991, 2009209992, 2010207181

Condition Reports:

2005100755, 2005111861, 2006107725, 2006107954, 2007103999, 2007104654, 2007107381, 2007107382, 2009102485, 2010104089, 2010104488, 2010110046, 2010112583, 2010112610, 2010114078, 2011105288, 2011105306, 2011105329, 2011105382

Drawings:

D-175012, Sheet 1, HVAC and Filter P&ID control room and computer room - unit 1, Version 39.0

D-205012, Sheet 1, HVAC and Filter P&ID control room and computer room - unit 2, Version 40.0

Procedures:

FNP-0-SOP-56.1, Technical Support Center HVAC System, Version 11.0

NMP-ES-003, Life Cycle Management, Version 11.0

Work Orders:

2090655601, 2082085401, 2101706801, 2101954701, 2102072301, 2102912301

Section 1R15: Operability Determinations

Condition Reports:

2011103456, 2011105273, 2011105276, 2011105291, 2011105597, 2011106190, 2011106318, 2011106641, 2011106792, 2011107041, 2011107156, 2010118079

Documents:

FNP-0-11-002, Prompt Determination of Operability, Revision 1

Drawings:

D-175002, Sheet 1, CCW PI&D – unit 1, Version 48.0

D-175012, Sheet 1, HVAC and Filter P&ID control room and computer room - unit 1, Version 39.0

D-205012, Sheet 1, HVAC and Filter P&ID control room and computer room - unit 2, Version 40.0

D-177036, Sheet 1, Logic Diagram Diesel 1C Auto Start & Loading, Version 10.0

D-207187, Component Cooling Water Pump 2B, Version 18.0

U-176407, General Arrangement: Mark II Butterfly Valve, Version 1.0

U-176839, Limitorque Valve Control

Procedures:

FNP-0-SOP-56.1, Technical Support Center HVAC System, Version 11.0

FNP-2-STP-914, Auxiliary Building Battery Charger Load Test, Version 19.0

Work Orders:

1101154301, 2110928302

Section 1R18: Plant ModificationsDocuments:

C111053201, Equivalency Determination for Acopian 14 VDC Redundant Power Supply System
 DOEJ-FX1111087101-C001, Seismic III/I Evaluation of Temporary Modification 1111087101
 TM 1111087101, Temporary mounting and connection of a dual power supply for the 1A steam
 generator feed pump EH controller design modification

Procedures:

NMP-DP-001, Operation Risk Awareness, Version 12.0
 NMP-DP-001-GL01, Risk Assessment Worksheets, completed sheets for this modification

Work Orders:

1111087102

Section 1R19: Post Maintenance TestingCondition Reports

330880, 2011103456, 2010103897, 2011105597, 2011106190, 2010118079

Drawings:

D-207187, Sheet 1, Component Cooling Water Pump 2B, Unit 2, Version 18.0

Procedures:

FNP-1-STP-4.2, 1B Charging Pump Quarterly Inservice Test, Version 64.0
 FNP-1-STP-10.5, Emergency Boration Valve Quarterly Inservice Test, Version 6.0
 FNP-2-STP-24.1, 2A, 2B, and 2C Service Water Pump Quarterly Inservice Test, Version 57.0

Work Orders:

109250291, 1103216801, 1110513801, 1111109101, 2091146701, 2091146706, 2110928302,
 SNC 308519, SNC59979

Section 1R22: Surveillance TestingCondition Reports:

2010116324, 2010116901, 2010116975

Procedures:

FNP-1-STP-9.0, RCS Leakage Test, Version 49.0
 FNP-1-STP-16.1, 1A Containment Spray Pump Quarterly Inservice Test, Version 47.0
 FNP-1-STP-912.0, Reactor Coolant Pump Bus Reactor Trip Undervoltage Relay Test, Version
 21.0
 FNP-1-STP-912.1, Reactor Coolant Pump Bus Undervoltage TDAFW Start Relay Test, Version
 1.0
 FNP-1-STP-913.0, Reactor Coolant Pump Bus Reactor Trip Underfrequency Relay Test,
 Version 19.0
 FNP-2-STP-9.0, RCS Leakage Test, Version 45.0
 FNP-2-STP-201.19, Reactor Coolant System Q2B13TE0422B1, Q2B13TE0422B2,
 Q2B13TE0422B3, and Q2B13TE0422D Loop Calibration and Operational Test, Version
 56.0

Work Orders:

1080394801, 1080395101, 1080990101, 2081074401, 2111014701

Section 1EP2: Alert and Notification System TestingProcedures and Manual

FNP-0-EPP-1.0, Tone Alert Radio and Siren Operation, Version 4

FNP-0-EPP-1.1, FNP Alert and Notification System (ANS) Program, Version 5

FNP-0-EPP-1.2, ANS Testing and Maintenance, Version 5

Joseph M. Farley Nuclear Plant, 2011 Emergency Information Calendar

Southern Company Letter to Residence and FNP Tone Alert Radio Operating Instructions, Revision 1

Records and Data

ANS Quarterly Maintenance Check List and Test Records, November 2009 – March 2011

Selected Records of Silent, Full Cycle, and Growl ANS Testing, November 2009 – May 2011

CR 2010118218, NRC Identified Discrepancy for October 2009 ANS Performance Indicator, December 14, 2010

CR 2010112863, The 2010 Tone Alert Radio Walk-down for Farley Nuclear Plant, September 21, 2010

Section 1EP3: Emergency Response Organization (ERO) AugmentationProcedures

FNP-0-TCP-50.2, Emergency Planning Controlled Functional Position Qualification Requirements, Version 7

Emergency Response Organization Minimum Staffing and Augmentation Table

NMP-EP-111-001, Emergency Notification Communicator Instructions – Farley, Version 2.0

Records and Data

Selected Off Hours/Unannounced Call-In Drills, 4th Quarter 2009 – 2nd Quarter 2011

Off Hours/Unannounced Exercise Report, Staff Augmentation Required, October 7, 2009

Emergency Response Organization Duty Roster - current list

Selected Qualification Records and Rosters from Plateau Tracking System by Position and by Individual Emergency Responder

Section 1EP4: Emergency Action Level (EAL) and Emergency Plan ChangesProcedures

Joseph M. Farley Nuclear Plant Emergency Plan, Revision 52

NMP-EP-110, Emergency Classification Determination and Initial Action, Version 1.0

NMP-EP-110-GL01, FNP EALs - ICs, Threshold Values and Basis Version 2.0

NMP-EP-111, Emergency Notifications, Version 3.0

Records and Data

Joseph M. Farley Nuclear Plant – Units 1 & 2, Post TMI Requirements for the Emergency Operations Facility (EOF), May 19, 1981

NRC Letter to Alabama Power Company, Subject: Emergency Preparedness Appraisal, November 13, 1981

Change Packages for Plans and Procedures

- 10 CFR 50.54(q) Evaluation Change Package, FNP Meteorological Instrumentation, September 8, 2010
- 10 CFR 50.54(q) Evaluation Change Package, Modified ERO Position Descriptions and Functions in the TSC, April 22, 2011
- 10 CFR 50.54(q) Evaluation Change Package, NMP-EP-111-001, Communication Program for ERO Recall, February 21, 2011
- FNP-0-EIP-0.0, Emergency Organization, Version 26
- FNP-0-EIP-4.0, Health Physics Support to the Emergency Plan, Version 41
- FNP-0-EIP-7.0, Security Support to the Emergency Plan, Version 21
- FNP-0-EIP-8.3, Communications Equipment Operating Procedure, Version 15
- FNP-0-EIP-11.0, Handling of Injured Personnel, Version 23
- FNP-0-EIP-16.0, Emergency Equipment and Supplies, Version 59

Section 1EP5: Correction of Emergency Preparedness Weaknesses and DeficienciesRecords and Data

- Emergency Preparedness Drill Report, Crew 1, November 24, 2010
- Emergency Preparedness Drill Report, Crew 2, August 31, 2010
- Emergency Preparedness Drill Report, Crew 3, April 19, 2010
- Emergency Preparedness Drill Report/NRC Evaluated Exercise, Crew 1, January 25, 2011

Audits and Self-Assessments

- Fleet Oversight Audit of Plants Farley, Hatch, and Vogtle Offsite Emergency Preparedness Support, C-EP-2011, January 24 - March 15, 2011
- Fleet Oversight Audit of Emergency Preparedness, F-EP-2011, Log: FFO-2011-2, February 28, 2011
- Audit Number F-EP-2011, Checklist Number 19, Interfaces with Local Governments, February 8, 2011
- Audit Number F-EP-2011, Checklist Number 16, Exercises and Drills, February 22, 2011
- Audit Number F-EP-2011, Checklist Number 14, Medical and Public Health Support, February 3, 2011
- Audit Number F-EP-2011, Checklist Number 13, Radiological Exposure Control, February 15, 2011
- Audit Number F-EP-2011, Checklist Number 11, Accident Assessment, February 25, 2011
- Audit Number F-EP-2011, Checklist Number 10, Emergency Facilities and Equipment, February 18, 2011
- Audit Number F-EP-2011, Checklist Number 5, Emergency Response Support and Resources, February 2, 2011
- Audit Number F-EP-2011, Checklist Number 4, Corrective Actions/Other Issues, January 31, 2011
- F-FOA-QTR-2011-1, Fleet Oversight Assessment, Plant Farley 4th Quarter 2010 Performance Report, January 31, 2010

Condition Reports

- CR 2010118332, Evaluate Fleet Standard for Percent Fuel Over Temperature Value for MIDAS and Evaluate Procedures, December 16, 2010
- CR 2010118336, December 2010 Exercise Critique, Notification of Offsite Agencies, December 16, 2010

CR 2011101222, Fleet Oversight Deficiency: Individuals Concurrently Filling More than One ERO Position on ERO Crew, January 31, 2011
 CR 2011103917, NRC Informed U2 ERDS was Transmitting, March 24, 2011
 CR 2010203035, Clarification of "Potential Loss of RCS" Initiating Condition, April 23, 2010
 CR 2010102128, Control Room Failed to Classify Emergency Event during Drill, February 25, 2010
 CR 2010118341, Shift Manager Did Not Immediately Dispatch Fire Brigade during Drill Delaying Restoration of Heat Sink, December 16, 2010
 CR 2011100233, Federal Telephone System Discovered Inoperable, January 7, 2011
 CR 2010118337, TSC Air Conditioner Condenser Fan Blade Replacement, December 16, 2010
 CR 2010209255, Revise of TCP-50.2 for ERO Qualification, December 17, 2010

Section 40A1: Performance Indicator Verification 71151

Condition Reports

20101011183, 2010108120, 2011103701, 2011103820

Documents:

Farley Unit 1 and Unit 2 Consolidated Data Entry, MSPI Derivation Report, MSPI Heat Removal System, Unavailability and Unreliability Indexes, dated second quarter 2011
 Farley Unit 1 and Unit 2 Consolidated Data Entry, Emergency AC Power System, Unavailability and Unreliability Indexes, dated second quarter 2011
 Selected Unit 1 and Unit 2 Control Room Logs from March 2010 through March 2011
 NEI 99-02, Regulatory Assessment Performance Indicator Guideline, Revision 6

Procedures:

FNP-0-AP-54, Preparation and Reporting of NRC PI Data and NRC Operating Data, Version 14.0
 FNP-0-SYP-25.0, Mitigating System Performance Index Desktop Guide, Version 3.0

Records and Data

DEP Records from 4th Quarter 2010 through 1st Quarter 2011
 Selected Drill and Exercise Participation Records from 4th Quarter 2010 through 1st Quarter 2011
 Siren test data from 4th Quarter 2010 through 1st Quarter 2011
 Selected ERO Personnel Qualification and Participation Records
 Job Performance Measures, CRO-362D, Perform Actions As Directed by the Shift Manager/Emergency Director in the Control Room, Version 1